

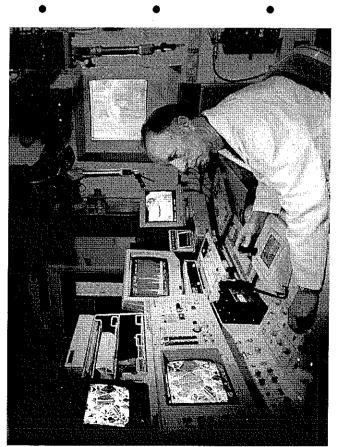
Developments in the use of SEM and SIMS for the study of Irradiated Fuel at Berkeley

• R C Corcoran and W J Stephen





Berkeley Shielded SEM



- JEOL 6100 FULLY SHIELDED (With analysis chamber in cell).
- EDX/WDX for chemical analysis.
- Adjacent cells for sample preparation.

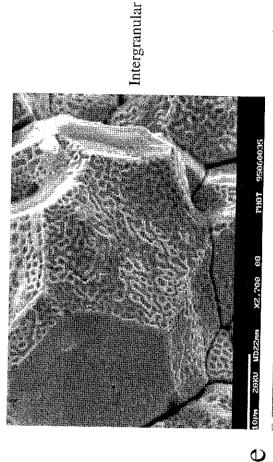


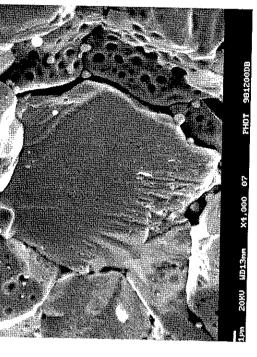
Fuel SEM



intragranular fission gas bubble Analysis of intergranular and populations

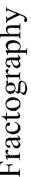
Distribution of fission gas related to position across fuel pellet.



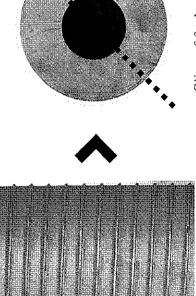




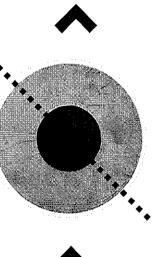
Fuel SEM Fractography



Fractured fuel slice.

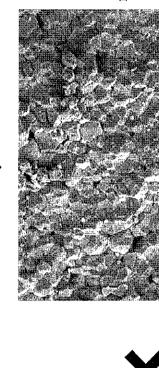


AGR Fuel Pin

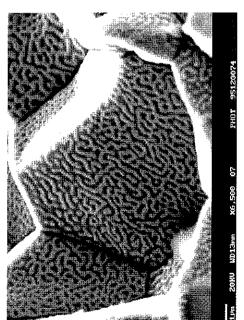


notched across diameter Slice of fuel cut and for fracture.

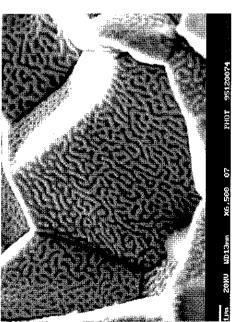
Position of inner bore



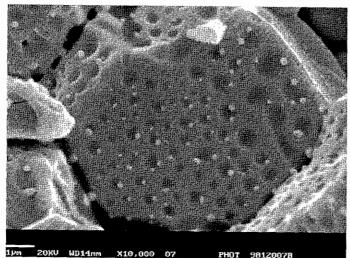
Low magnification image of fracture surface

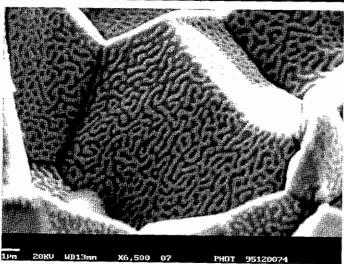


High magnification image of intergranular facet



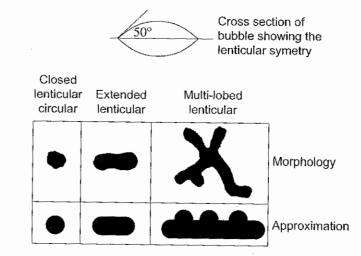






Intergranular Volume Swelling

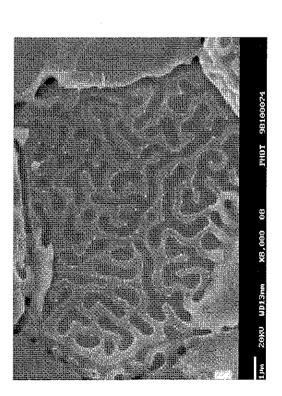
- Depending on Irradiation History bubbles form:
 - Circular Lenticular Bubbles
 - Extended Lenticular Bubbles
 - Multi-lobed Lenticular Bubbles



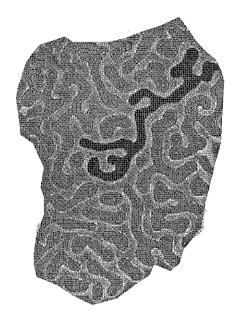


Intergranular Volume Swelling

In-House Software Developed for Image Analysis



SEM Image of grain face



-Bubbles Outlined Grain area extracted

- Measure:
 -Perimeter (P)
 -Areas (A)
 -Length (L)
 -Number of nodes (n)

Get Volume (V_{n1})



Intergranular Volume Swelling

In-House Software Developed for Image Analysis

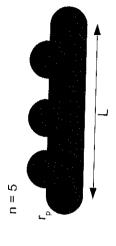
Skeletonised Feature Determine L and n

Extracted Feature Measure A and P

Model of Extended Pore







$$P = 2L + (n\pi + 4 - 2n)r_p$$

$$A = \frac{n}{2}\pi r_p^2 + 2Lr_p$$

$$V_{nl} = n \frac{2\pi}{3} \left(\frac{r_p}{\sin \theta} \right)^3 f_f(\theta) + L \left(\frac{r_p}{\sin \theta} \right)^2 f_p(\theta)$$

L, r_p and n allow bubble morphology to be examined



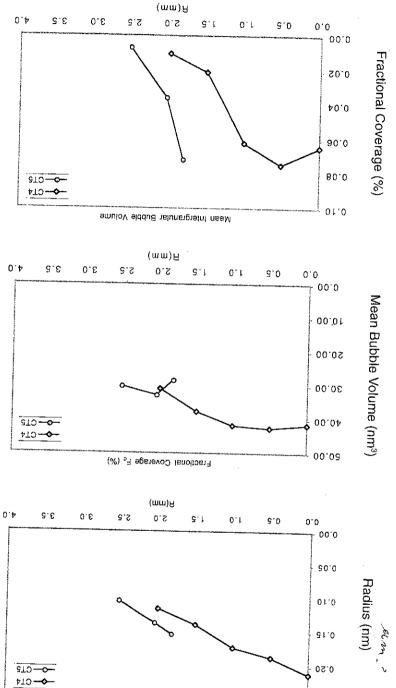
Berkeley Fuel TEM



- Phillips CM12 Dedicated Fuel TEM
- EELS for thickness measurements
- EDX for chemical analysis
- Analysis of small intra-granular fission gas bubbles (<50nm diameter)
- General microstructural examination of fuel

Magnox Ceneration

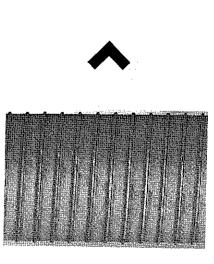
BNLF



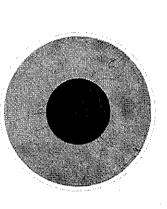
Mean Infergranular Lenticular Bubbie Radii $(\mathfrak{r}_{\mathfrak{p}})$

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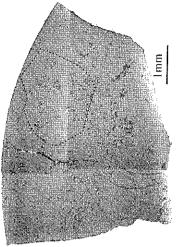




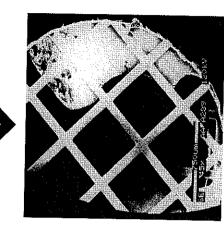
AGR Fuel Pin



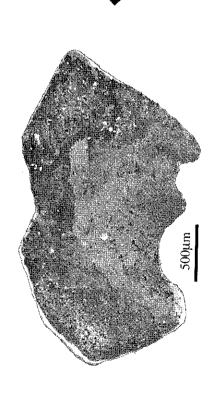
Slice of fuel pin cut



Segment of fuel examined in \$SEM

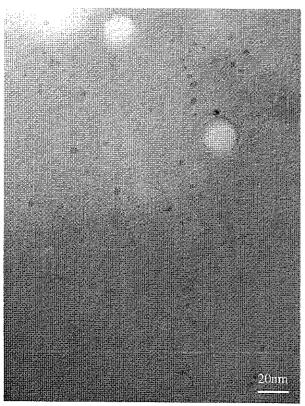


Segment of fuel electropolished and examined in TEM

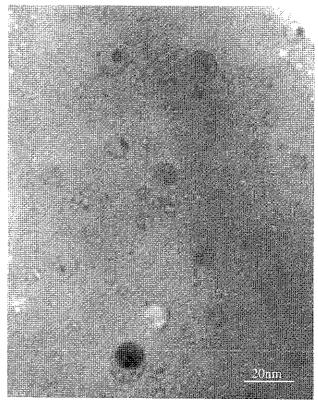


TEM sample examined in SSEM to locate analysis position



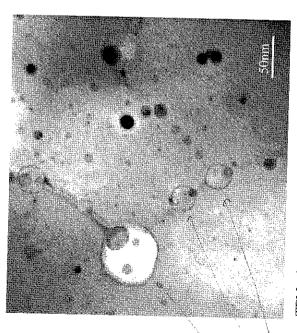


TEM micrograph taken over-focus of 4065 (slow ramp + scram) at 4.1 mm from the clad. The fission gas bubbles image dark, are faceted and have nucleated in lines.



TEM micrograph taken under-focus in 4065 (slow ramp + scram) at 1.45 mm from the clad showing the bimodal fission gas bubble distribution with the larger population all having a solid fission product precipitate inside.





TEM micrograph of sample as in Figure 3(b), showing associated with both solid fission product precipitates the 3rd and larger population of fission gas bubbles and distocations.

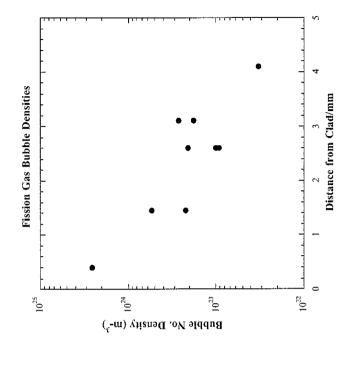


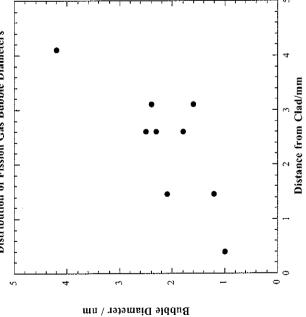
mm from the clad showing the large fission gas bubbles which are those observed by the scanning electron TEM micrograph of 4065 (slow ramp + scram) at 3.1 microscope.



(AGR Fuel ramped in Halden)

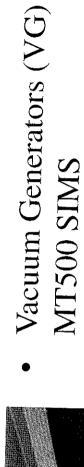


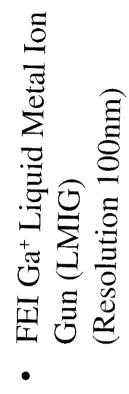




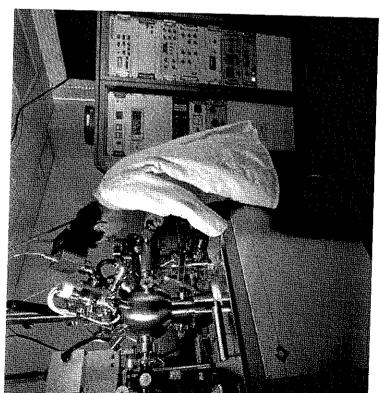


Berkeley SIMS





VG EX05 Ar⁺ Ion Gun (Resolution 30µm)





Advantages for Fuel Analysis:

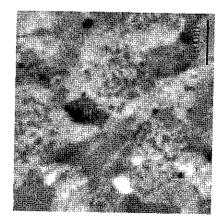
- Isotopic sensitivity e.g. can map/profile ²³⁹Pu or ²⁴⁰Pu in MOX
 (Does not have the same peak overlap problems associated with EDX/WDX)
- Increased spatial resolution, 0.1µm relative to large 1µm X-ray generation volumes in EDX/WDX
- Allows depth information to be acquired Depth profiles used to determine Diffusion Coefficients

Disadvantages:

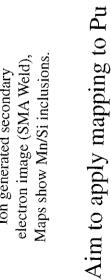
- Destructive analysis erodes specimen
- Difficult to quantify
- Requires expensive UHV equipment
- Very surface sensitive







electron image (SMA Weld), Ion generated secondary

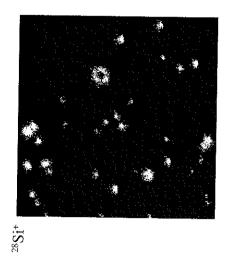


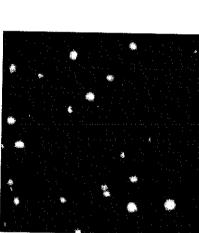


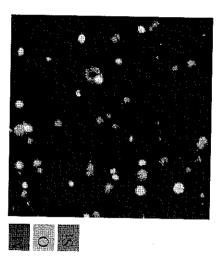
irradiated and unirradiated

MOX fuel.

isotope distributions in

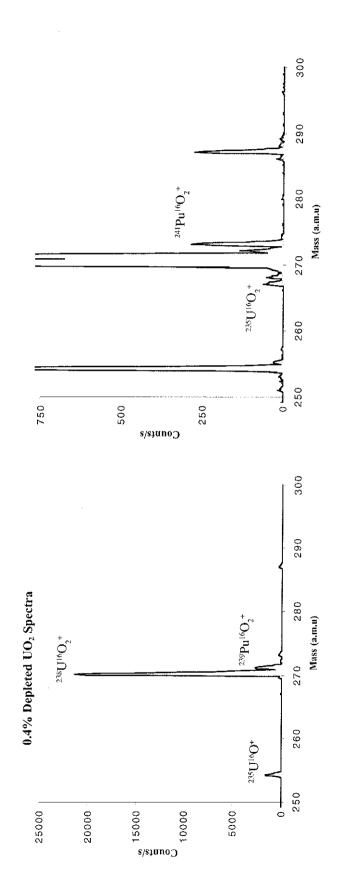






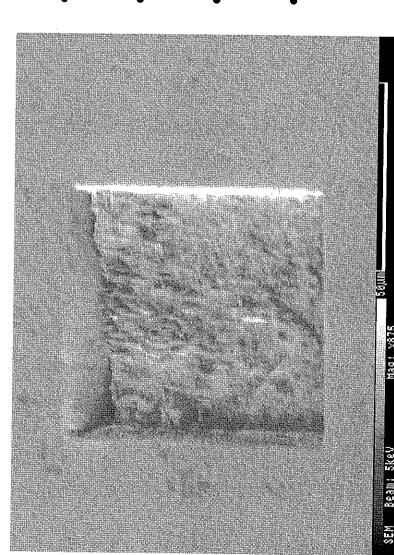


Isotopic Analysis of Fuel - Used to generate isotopic radial profiles





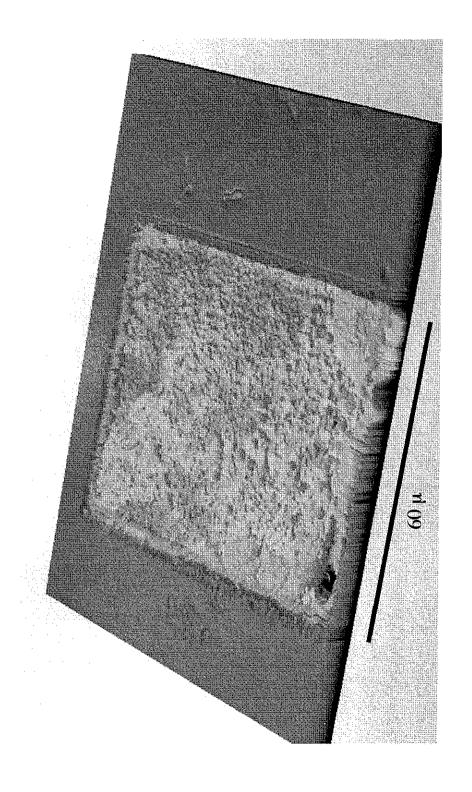
Depth Profiles - Iodine Diffusion in Uranium



- Ga⁺ ion gun crater
- 90mm x 60mm
- 6.6µm deep
- roughness 0.5µm Crater surface



Optical Profilimetry of Crater in Uranium



 Iodine concentration measured as a function of depth

Use Fick's Diffusion Laws to obtain Diffusion Coefficient

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1500

1000

Sounts

200

lodine Profile

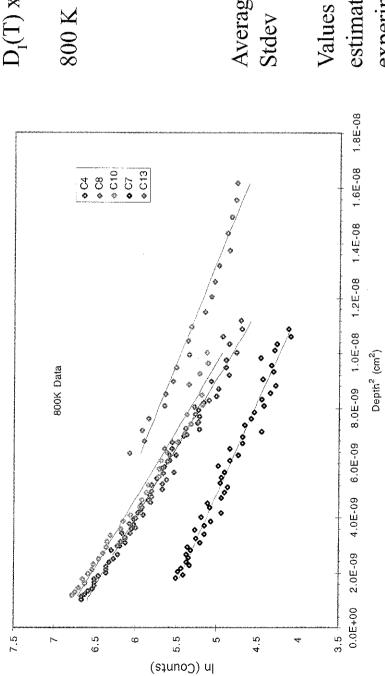
$$c(z,t) = c_o \exp\left(\frac{-z^2}{4Dt}\right)$$

$$\ln[c(z,t)] \ vs \ z^2$$

Depth (µm)

$$Gradient = \frac{-1}{4Dt}$$





 $D_{\rm I}(T) \text{ x} 10^{-15} \text{cm}^2 \text{s}^{-1}$

 $800 \, \mathrm{K}$

0.73 0.97 1.07 0.72 0.76

Average

0.85

Values agree with previous estimates from Xe release experiments



Summary

- Quantitative Fission Gas analysis has been applied extensively to AGR UO₂ and now MOX Fuel.
- SIMS now being applied to MOX to:
- Produce 239Pu maps on unirradiated/irradiated fuel
- Produce radial profiles on irradiated fuel.

