



# Examination of analytical method of rare earth elements in used nuclear fuel



**B**ack-**E**nd fuel **C**ycle **K**ey  
elements research facility**Y**

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# 1. Background (1/2)

- For criticality safety evaluation,
  - The burnup value is important and indispensable.
  - Burnup calculation codes are being continuously improved or developed.
  - It is required to evaluate the amounts of fission products in used nuclear fuel.



- Some isotopes of rare earth elements such as Gd, Eu and Sm have large neutron absorption cross sections and high fission yield.
- The amounts of these isotopes are computed by burnup calculation codes for criticality safety evaluation.



Some isotopes of rare earth elements have same mass number.

## Objective

To separate rare earth elements prior to the measurement by mass spectrometry.

# 1. Background (2/2)

- For the evaluation of burnup,
  - JAEA had developed an analytical method to measure amounts of U, Pu and Nd.

“Conventional method”



We examine to measure the rare earth elements such as Gd, Eu and Sm **by improving the conventional method.**

- To obtain isotopic composition data of used nuclear fuel **including the rare earth elements in addition to U, Pu and Nd** without complicated process added.
- To obtain these data from **one sample.**

## 2. Improvement of the analytical method

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### 1. Background

### 2. Improvement of the analytical method

- 2.1 Conventional separation scheme
- 2.2 Condition of separation experiment
- 2.3 Result

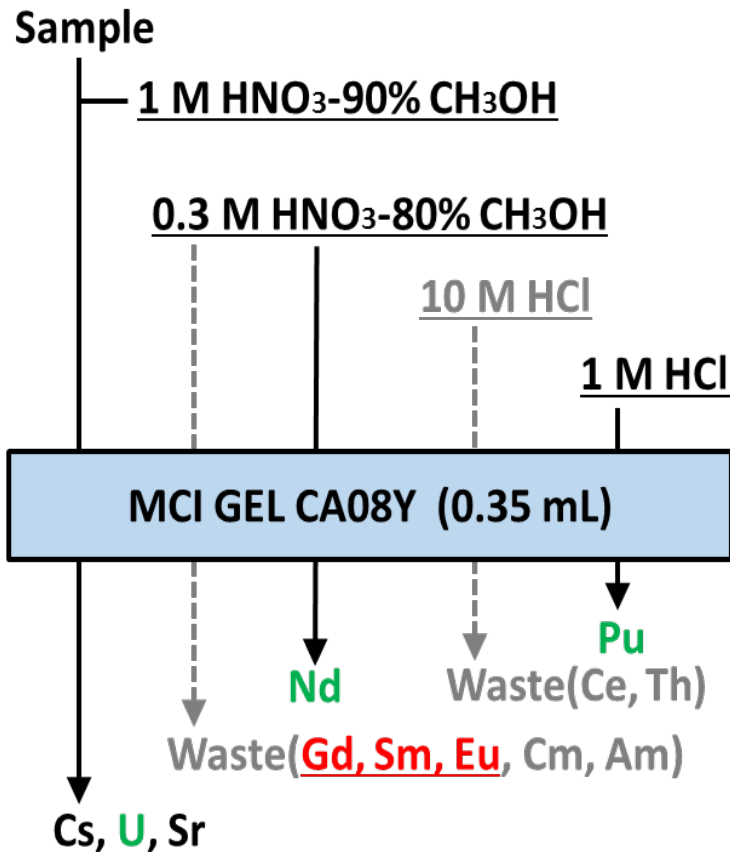
### 3. Evaluation of the improved analytical method

- 3.1 New separation scheme
- 3.2 Confirmation of new scheme
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### 4. Conclusion

## 2.1 Conventional separation method

- Quartz column; inner diameter of 3mm
- Anion exchange resin; MCI GEL CA08Y, volume of 0.35 mL
- Temperature; 26°C during elution with thermostatic bath



### ➤ Conventional method

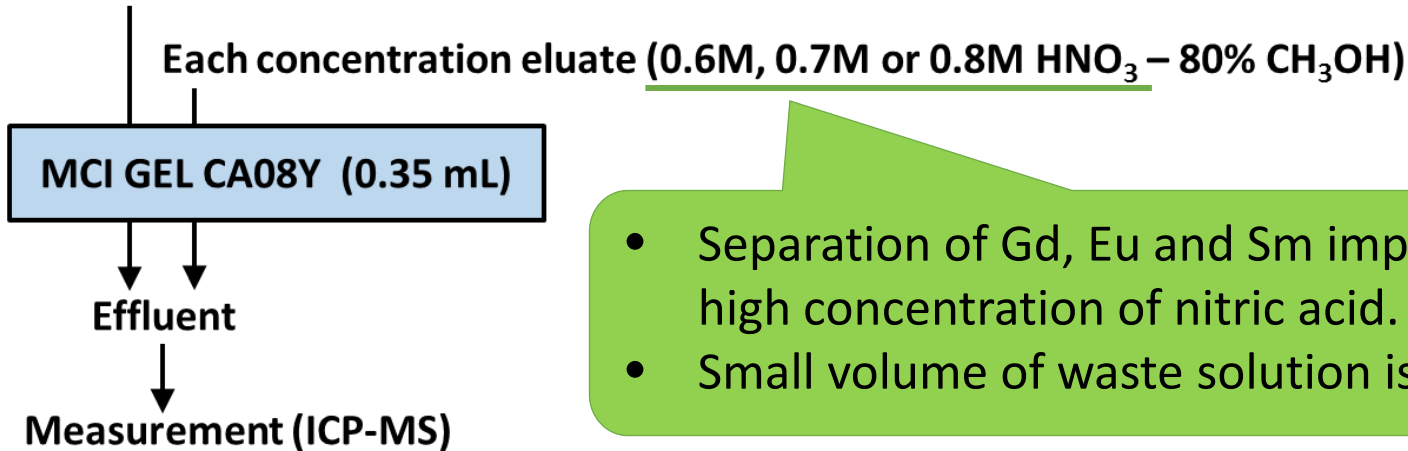
- Suitable method to separate **U, Pu** and **Nd**.
- Rare earth elements such as **Gd, Eu** and **Sm** except Nd are washed out to waste portion.



We examined to separate these elements in waste portion by adding another eluate of different chemical composition.

## 2.2 Experimental condition of separation

Sample (stable isotopes)



- Separation of Gd, Eu and Sm improves with high concentration of nitric acid.
- Small volume of waste solution is also required.

Amounts of samples	<sup>149</sup> Sm : about 200 pg <sup>151</sup> Eu : about 150 pg <sup>155</sup> Gd : about 350 pg
Eluate concentrations	1) 0.6 M HNO <sub>3</sub> -80% CH <sub>3</sub> OH 2) 0.7 M HNO <sub>3</sub> -80% CH <sub>3</sub> OH 3) 0.8 M HNO <sub>3</sub> -80% CH <sub>3</sub> OH
Feed sample condition	HNO <sub>3</sub> 0.4 mL + CH <sub>3</sub> OH 4 mL
Temperature	26°C
Analytical equipment	<u>I</u> nductively <u>C</u> oupled <u>P</u> lasma <u>M</u> ass <u>S</u> pectrometry (ICP-MS)

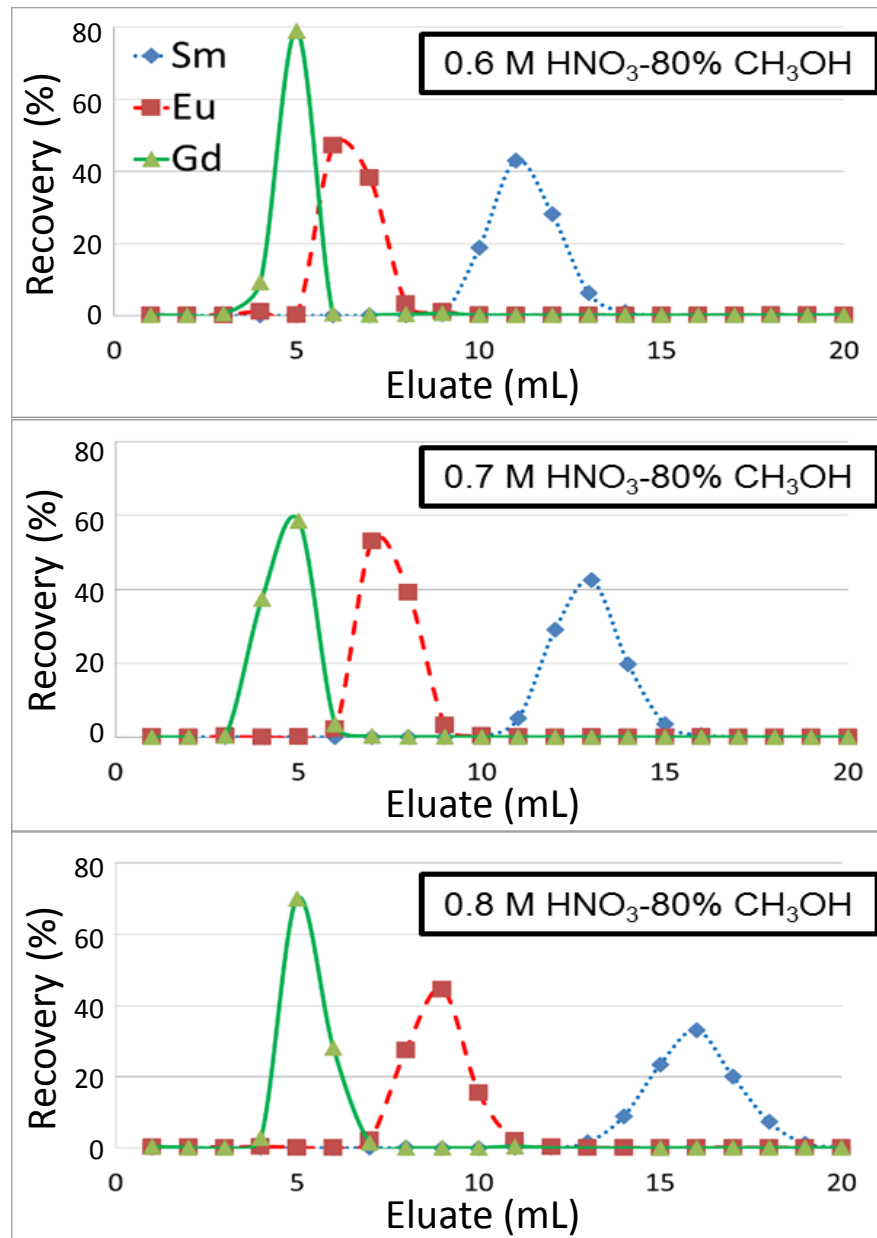
## 2.3 Result

- Separation of Gd, Eu and Sm is enough with 0.8 M HNO<sub>3</sub> - 80% CH<sub>3</sub>OH.
- Total volume of eluate is acceptable all cases.



We selected

0.8 M HNO<sub>3</sub> - 80% CH<sub>3</sub>OH among three different concentrations for the separation of Gd, Eu and Sm.





## 3. Evaluation of the improved analytical method

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### 1. Background

### 2. Improvement of the analytical method

- 2.1 Conventional separation scheme
- 2.2 Condition of separation experiment
- 2.3 Result

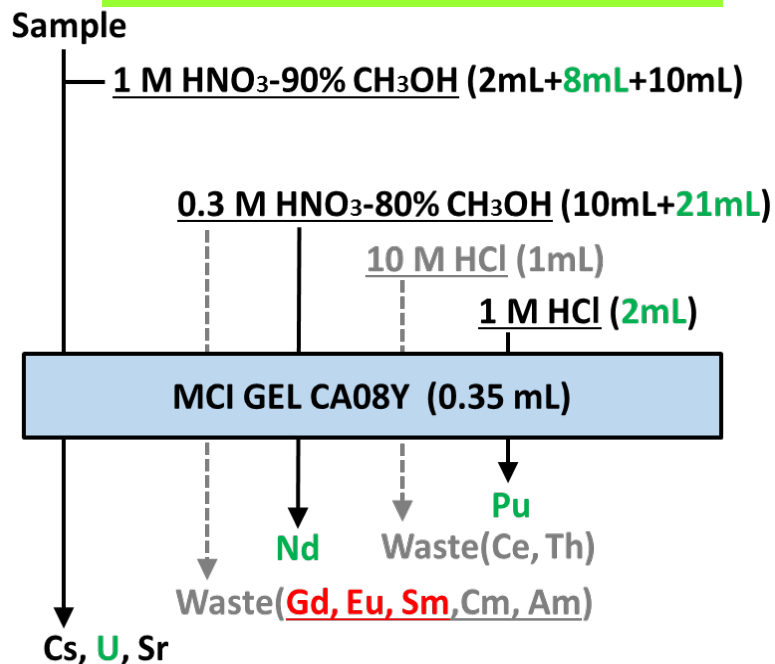
### 3. Evaluation of the improved analytical method

- 3.1 New separation scheme
- 3.2 Confirmation of new scheme
- 3.3 Result

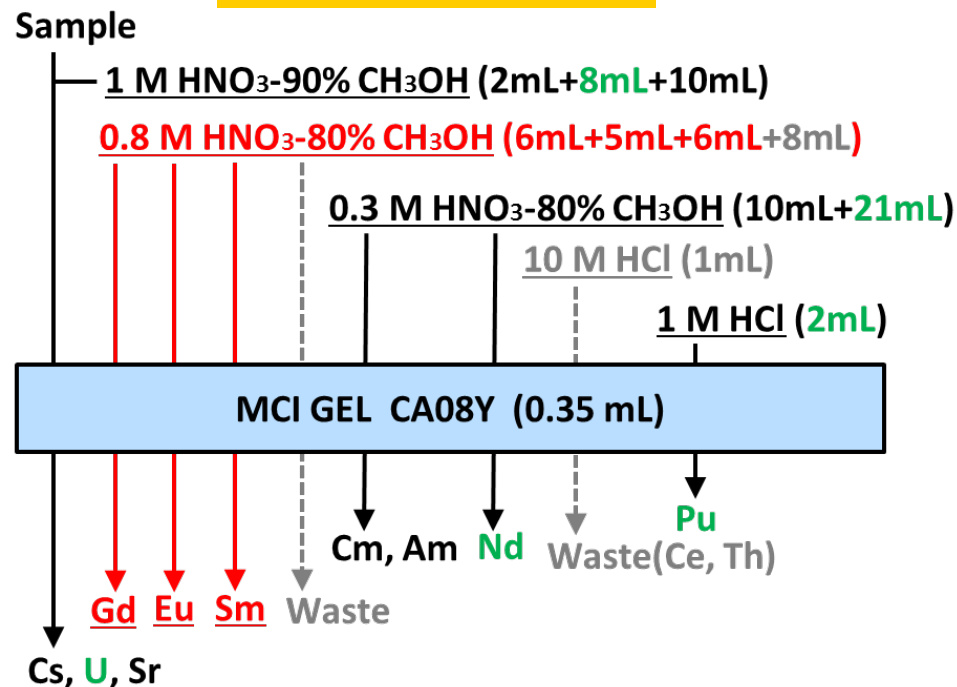
### 4. Conclusion

# 3.1 New separation scheme

## <<Conventional scheme>>



## <<New scheme>>



The separation process of Gd, Eu and Sm with 0.8M HNO<sub>3</sub>-80% CH<sub>3</sub>OH is added between the elutions of U and Nd.

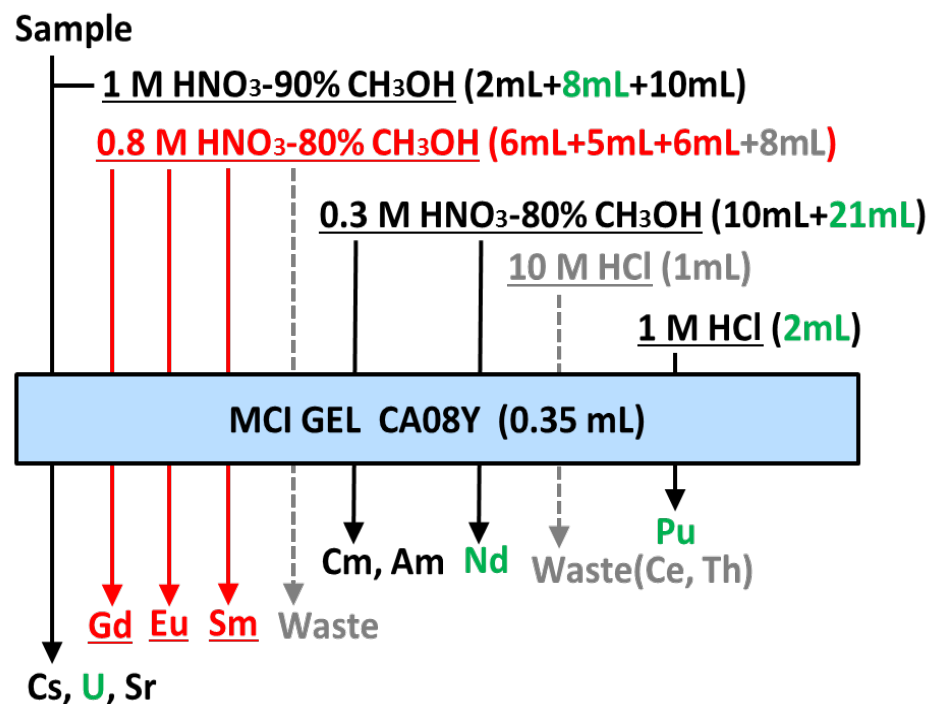


We confirm the influence of the added process on elution of Nd and Pu.

## 3.2 Confirmation of new scheme

- U, Pu and Nd were used in order to confirm the influence of the newly added process.
- Amounts of these elements are about the same with the amounts included in the used nuclear fuel samples.

Amounts of samples	$^{233}\text{U}$ : about 2 $\mu\text{g}$
	$^{242}\text{Pu}$ : about 50 ng
	$^{150}\text{Nd}$ : about 20 ng
	$^{149}\text{Sm}$ : about 200 pg
	$^{151}\text{Eu}$ : about 150 pg
	$^{155}\text{Gd}$ : about 350 pg
Feed sample condition	$\text{HNO}_3$ 0.4 mL + $\text{CH}_3\text{OH}$ 4 mL
Temperature	26°C
Analytical equipment	ICP-MS



## 3.3 Result (1/2)

Fractions	Recovery (%)							
	The conventional method				The new method			
	$^{233}\text{U}$	Gd, Eu, Sm	$^{150}\text{Nd}$	$^{242}\text{Pu}$	$^{233}\text{U}$	Gd, Eu, Sm	$^{150}\text{Nd}$	$^{242}\text{Pu}$
U fraction	71.4	< 1	< 1	< 1	74.1	< 1	< 1	< 1
Nd fraction	< 1	< 1	97.7	29.8	< 1	< 1	96.4	36.6
Pu fraction	< 1	< 1	< 1	62.9	< 1	< 1	< 1	52.9

- Elements which interfere with the measurement of the target did not elute in each fraction.
- Recovery rates of U, Pu and Nd are almost same between the conventional method and the new method.
- ➔ The newly added separation process did not interfere with separation and measurement of U, Pu and Nd.

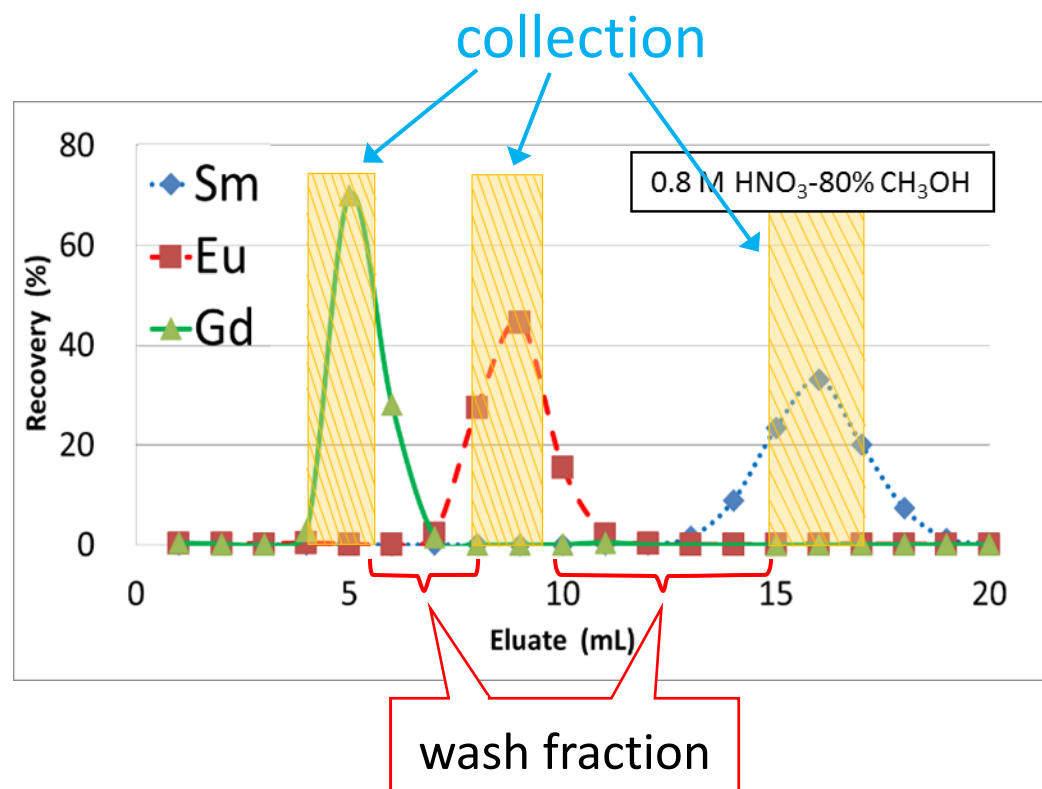
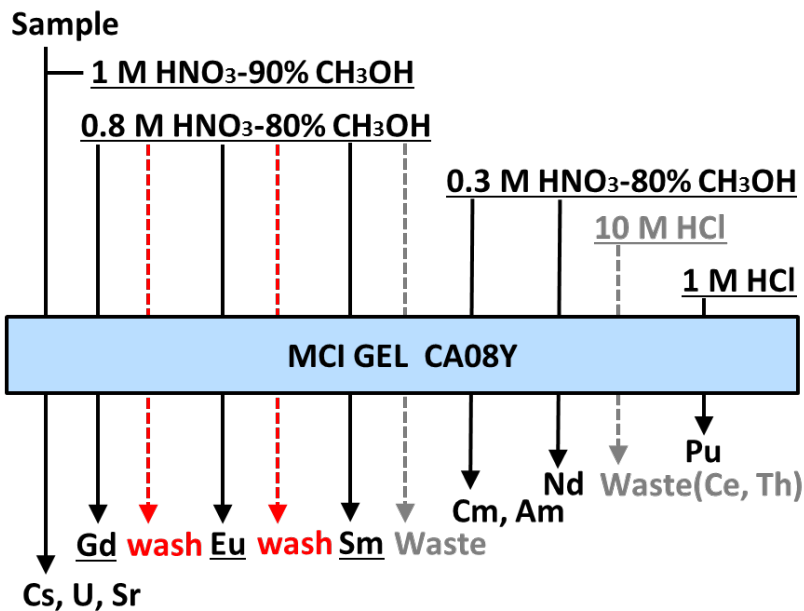
### 3.3 Result (2/2)

Fractions	Recovery (%)		
	$^{155}\text{Gd}$	$^{151}\text{Eu}$	$^{149}\text{Sm}$
Gd fraction	93.0	25.1	< 1
Eu fraction	4.4	77.9	1.3
Sm fraction	< 1	< 1	91.4

- The new method succeeded to separate most of Gd, Eu and Sm.
- Eu eluted in Gd fraction, and Sm and Gd eluted in Eu fraction.
  - High recovery is not necessary, but high purity is required for the measurement of each element by IDMS method.

# 《Future work》

- Sampling near the boundary of each element should be avoided, especially before and after Eu fraction.
- It is required to collect effluent near the peaks of each element for the measurement with highly pure samples.



## 4. Conclusion

- We obtained prospect for separation of Gd, Eu and Sm by the new method.
- The added separation process of rare earth elements did not interfere with separation of U, Pu and Nd.



- The new method made it possible to obtain isotopic composition data of used nuclear fuel including the rare earth elements.
- The data obtained by the new method will be used for:
  - Validation and improvement of burnup calculation codes.
  - Burnup credit calculation of used nuclear fuel.