



Poster session

Wade Karlsen, VTT

1 slide in 2 min each

P01

Magnus Göhran

*European Spallation Source ERIC, Fagerström
Industrikonsult AB*

The European Spallation Source Active Cells Facility -
Challenges in Construction - HOTLAB 2018

- Mechanical versus civil construction tolerances
 - 2 different worlds meet at the same location
- Re-bar design and casting sequences
 - 500 kg/m³ re-bar, 1.3 m thick high density concrete walls
- Uneven concrete surfaces
 - Casting formworks shift and swell differently during casting
- Installation of cast in items in tight spaces
 - Clashes between cast in items and formworks



P02

Alex Wagner

United Kingdom Atomic Energy Authority

Size Reduction Equipment in the ESS Active Cells
Facility

ESS ACF Size Reduction Equipment

Alex Wagner – UK Atomic Energy Authority

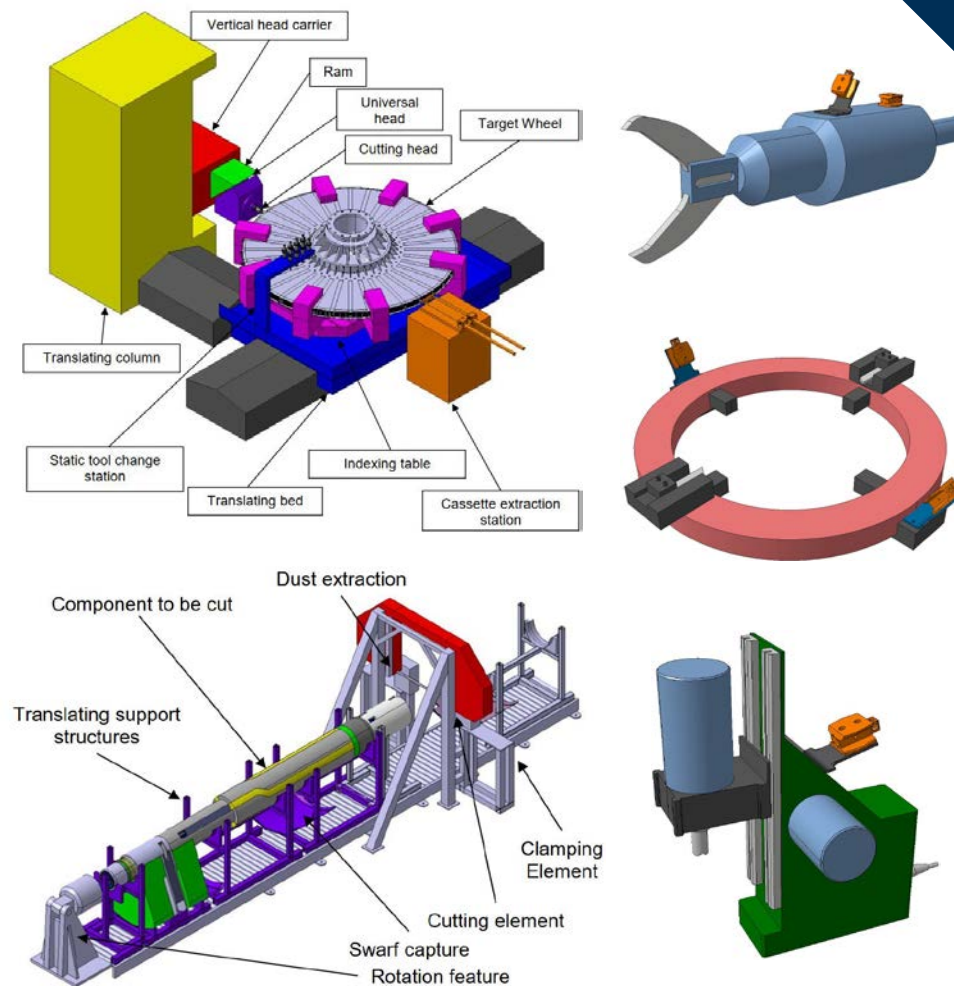
A variety of large radioactive components shall require size reduction and handling operations within the European Spallation Source (ESS) Active Cells Facility.

A systems engineering approach was conducted, producing a function-based set of requirements for various size reduction equipment. Size reduction equipment includes:

1. Machining Station (accepting tender submissions)
2. Shaft Cutting Station (tenders in review)
3. Auxiliary tools (shear cutter, orbital cutter, bolt removal tools)

Main difficulties of cutting technologies required:

1. Radiation hardness and remote maintainability
2. Dry cutting (no coolant liquids)
3. Large cut depths and long cut paths





P03

Mika Helin

Platom Oy

PLATOM's Expertise and Capabilities to Support
Construction and Operation of Hot Cell Facilities

PLATOM's Expertise and Capabilities to Support Construction and Operation of Hot Cell Facilities

- Platom has 20 years of experience in delivering consultancy services, components and systems for nuclear industry in several countries
- The evaporation and waste treatment (EWT) box for radioactive waste water treatment from Hot Cell operations was delivered to VTT in spring 2018
- Earlier a drying cabinet for liquid waste volume reduction was supplied to Forsmark NPP
- These deliveries further expand the services Platom can offer to the international nuclear industry



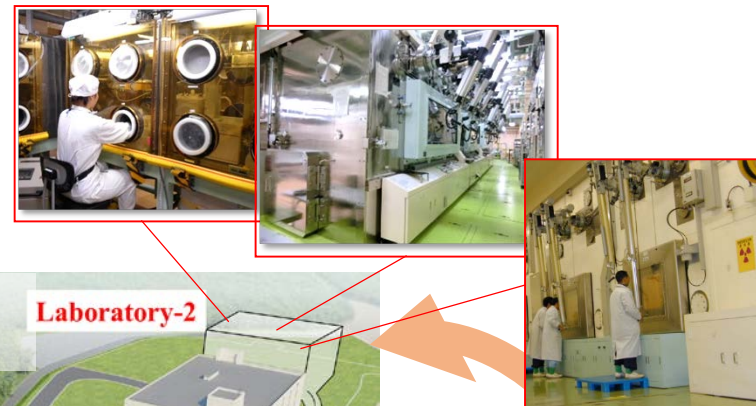
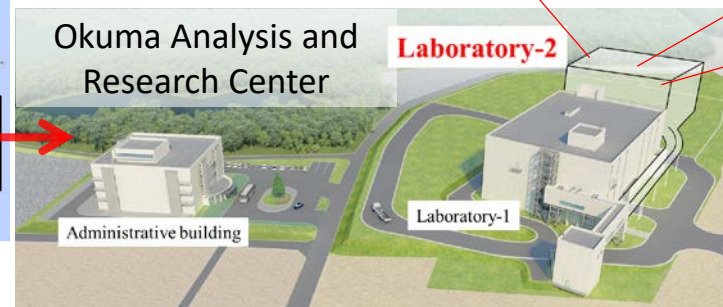
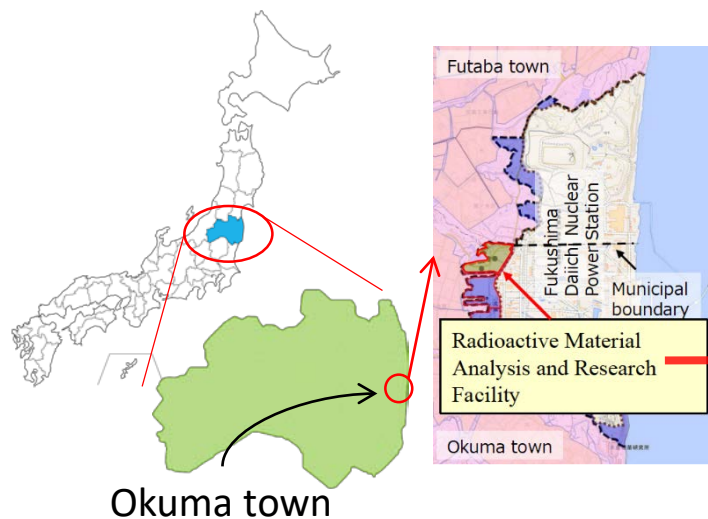
P04

Akinori Sato

Japan Atomic Energy Agency

The preliminary study for safety design of JAEA's Radioactive Material Analysis and Research Facility "Laboratory-2" dedicated to fuel debris analysis at TEPCO's Fukushima Daiichi Nuclear Power Station site

- In 2011 May, 1F accident occurred.
- According to Mid-and-Long-Term Roadmap for decommissioning of 1F NPS, fuel debris retrieval from the first unit will start in 2021.



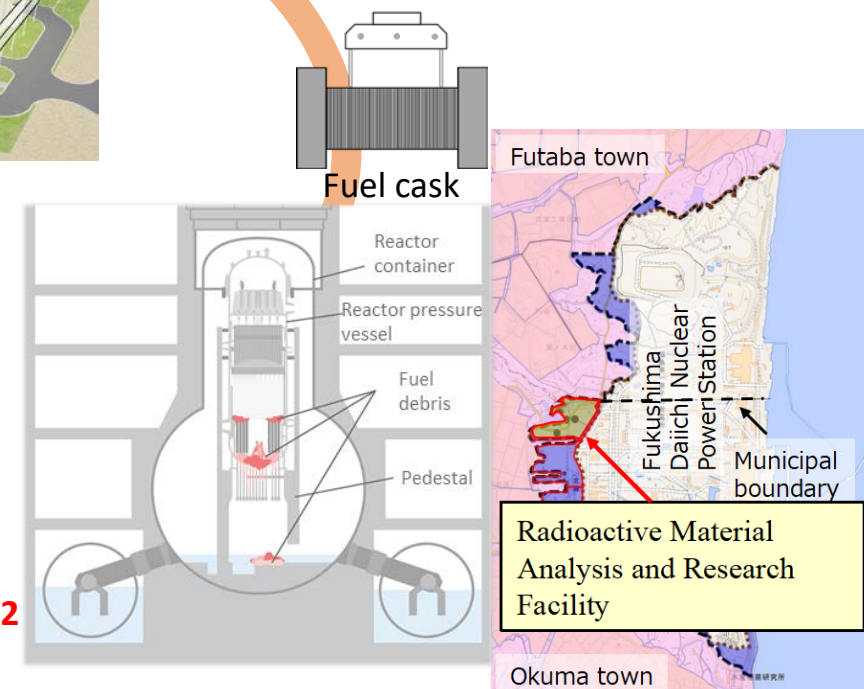
- JAEA is designing the Radioactive Material Analysis and Research Facility (**Okuma Analysis and Research Center**) adjacent to the 1F site.

• In **Laboratory-2**, fuel debris is mainly handled and analyzed to **characterize chemical and mechanical property**.

- The practically conservative assumption is requested for the safety design.

The radiation shielding evaluation and the criticality safety evaluation

- Through the preliminary evaluations, **the feasibility of safety design of Laboratory-2 is implied**.





P05

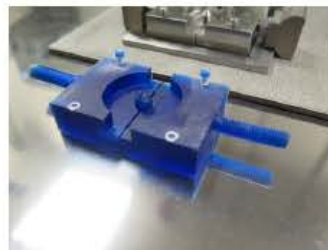
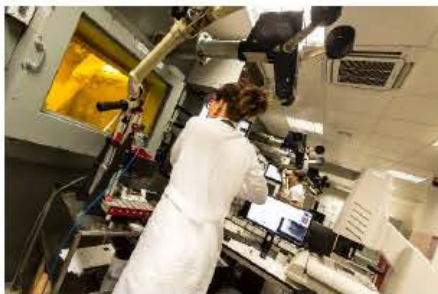
Ondrej Srba

Research Centre Rez Ltd.

Preparation of experiments at CVR Hot-cell

Preparation of experiments at CVR Hot-cell

Ondřej Srba



Specimen



Replica



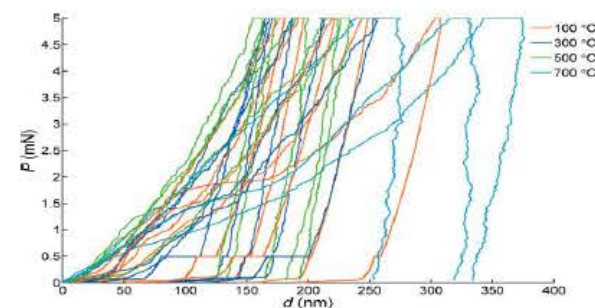
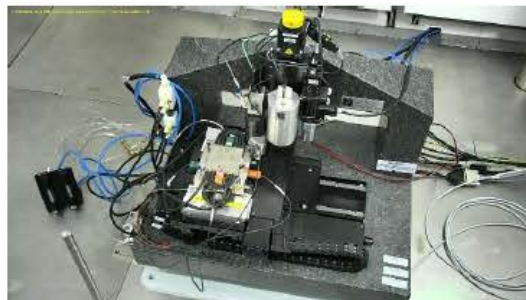
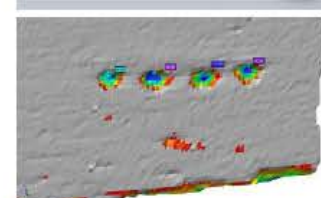
3D Model



Making replica



Scanning





P06

Laurent Velnom

*European Commission, Joint Research Centre,
Directorate for Nuclear Safety and Security*

An overview of the Remote Handling solutions and
equipment at JRC Karlsruhe's Hot Cells facilities

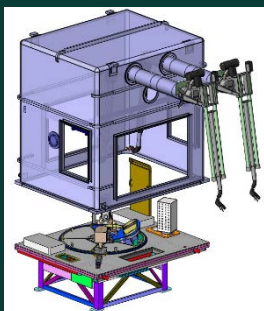
An overview of the Remote Handling solutions and equipment at JRC Karlsruhe's Hot Cells facilities

L. Velnom, A. Busto, J. Marconato

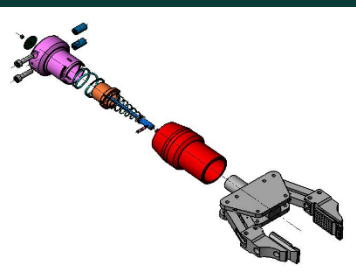
European Commission, Joint Research Centre, Directorate for Nuclear Safety and Security,
Postfach 2340, D-76125 Karlsruhe, Germany



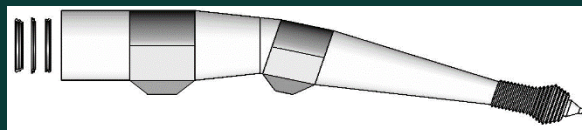
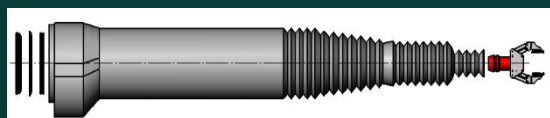
The Radiochemistry Hot Cells



High Activity α steel cases with its two MSM'S .



R&D on Grippers



Various types of engineered bootings



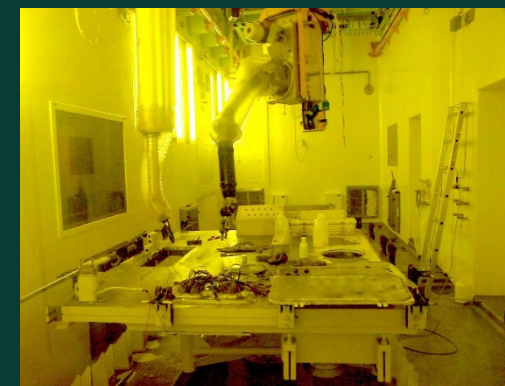
Minor Actinide Laboratory



Intervention on MA Lab HC



Heavy duty SAMM power manipulator



MA 23 with force feedback in Decontamination HC

P07

[Patrik Sandström](#)

Studsvik Nuclear AB

Self-threading electrical discharge machine - hot
cell modifications and the first year of active
machining

P08

V. Revka

Institute for Nuclear Research

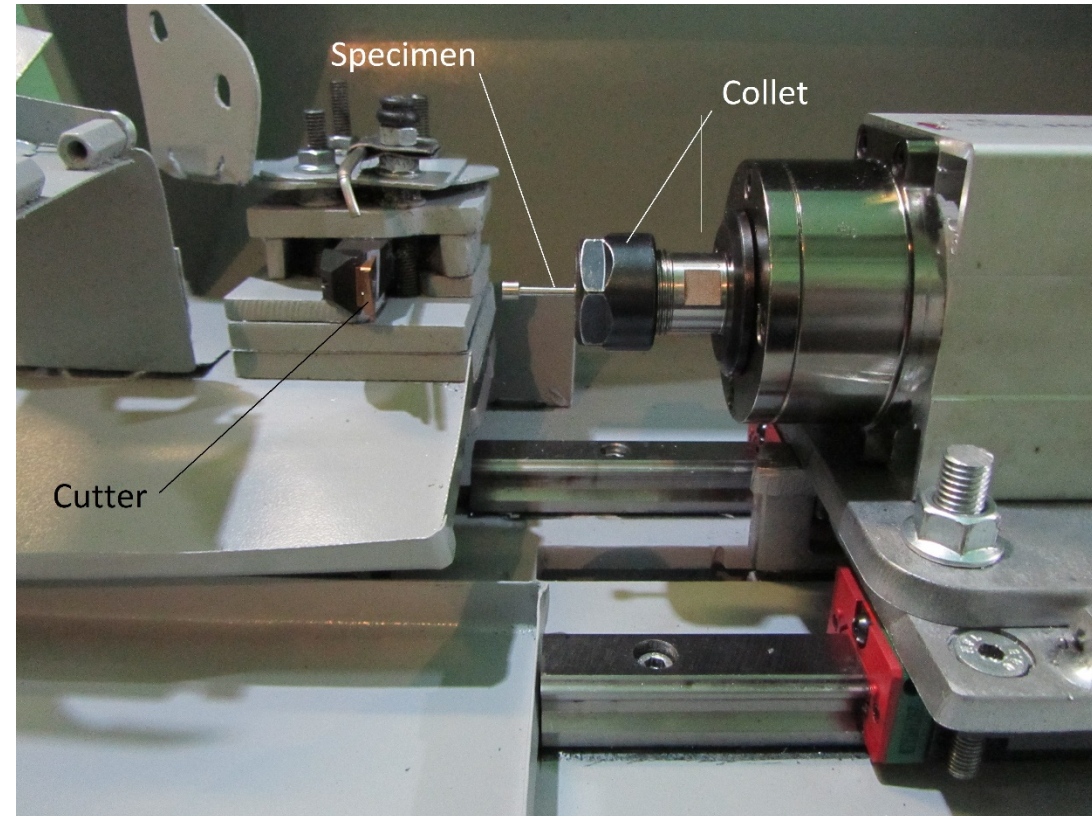
A remote technique for a preparation of tension
test specimens from the irradiated round bars

A remote technique for a preparation of tension test specimens from the irradiated round bars (P08)

Volodymyr Revka

Institute for Nuclear Research, Kyiv, Ukraine

- ✓ Round bars have been put in the surveillance capsules instead of standard tension specimens (Rivne NPP-1)
- ✓ Preparation of tension specimens from irradiated round bars is needed
- ✓ A remote lathe with computer control has been developed
- ✓ A brief description of equipment
- ✓ Application of equipment for machining specimens





P09

Cameron Howard

Canadian Nuclear Laboratories (CNL)

Using Novel Small Scale Mechanical Testing to Link
the Mechanical Properties and Deformation
Mechanisms of High-Dose Activated Inconel X-750

Using Novel Small Scale Mechanical Testing to Link the Mechanical Properties and Deformation Mechanisms of High-Dose, Activated Inconel X-750



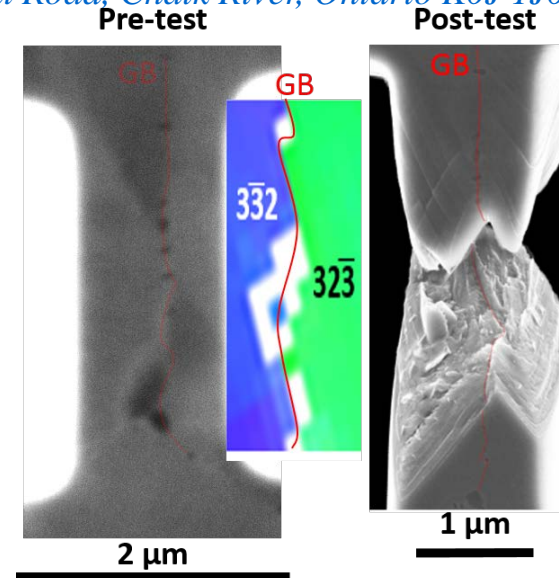
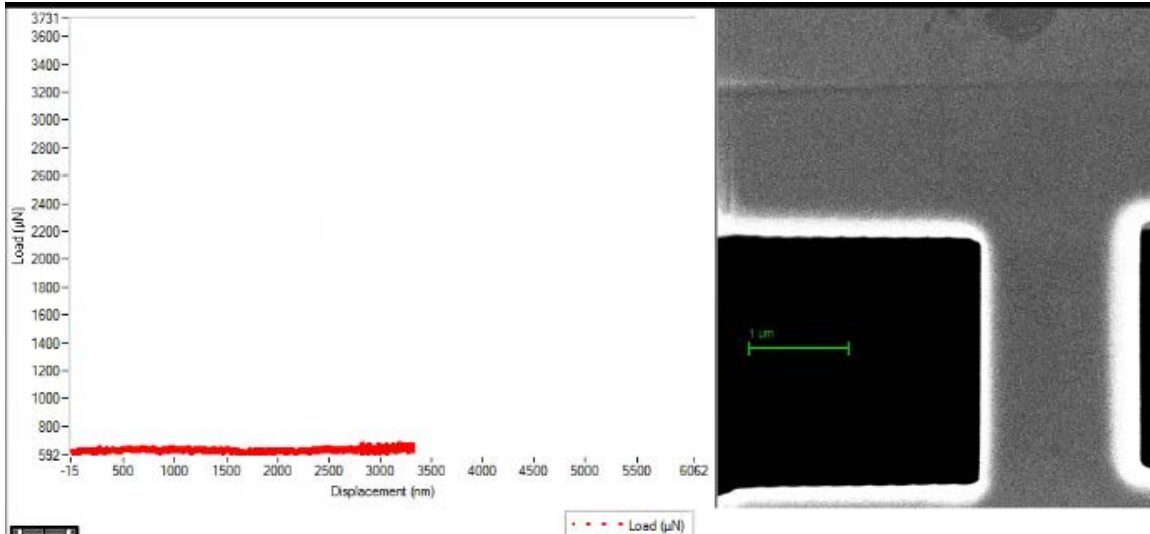
Canadian Nuclear
Laboratories

Laboratoires Nucléaires
Canadiens

Cameron Howard, Vineet Bhakhri, Chris Dixon, Heygaan Rajakumar, Clinton Mayhew, Colin Judge

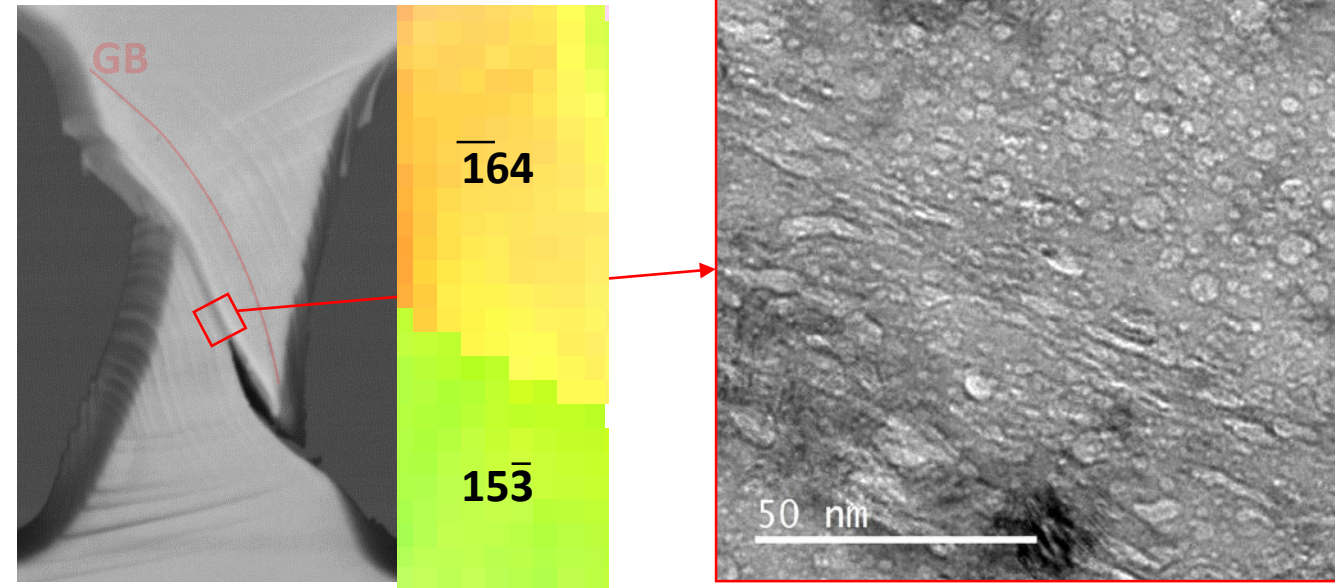
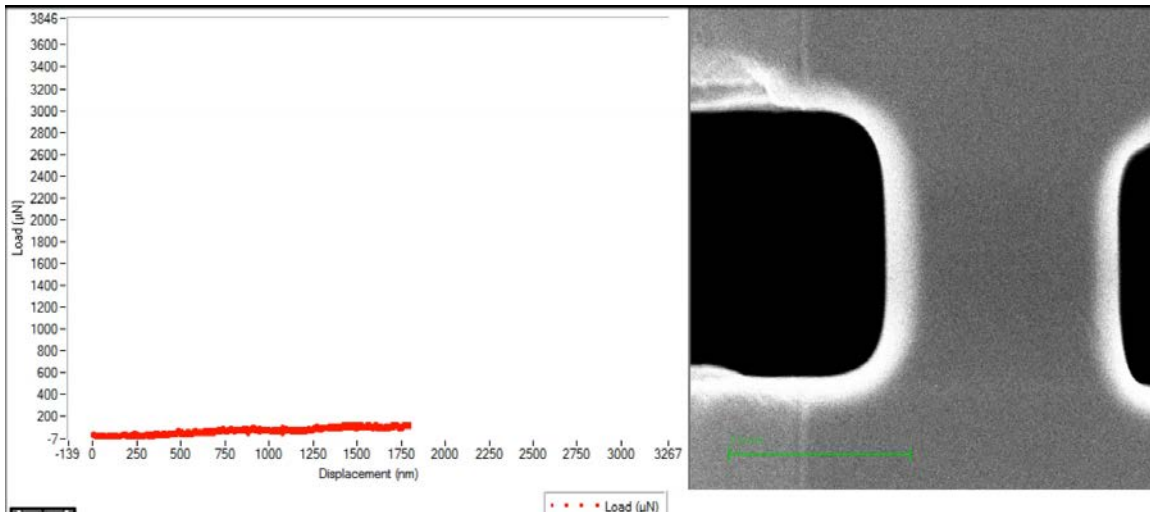
Canadian Nuclear Laboratories (CNL), 286 Plant Road, Chalk River, Ontario K0J 1J0, Canada

~20 dpa, $T_{irr} = 300\text{ }^{\circ}\text{C}$



intergranular fracture

84 dpa, $T_{irr} = 300\text{ }^{\circ}\text{C}$



trans-granular channel fracture



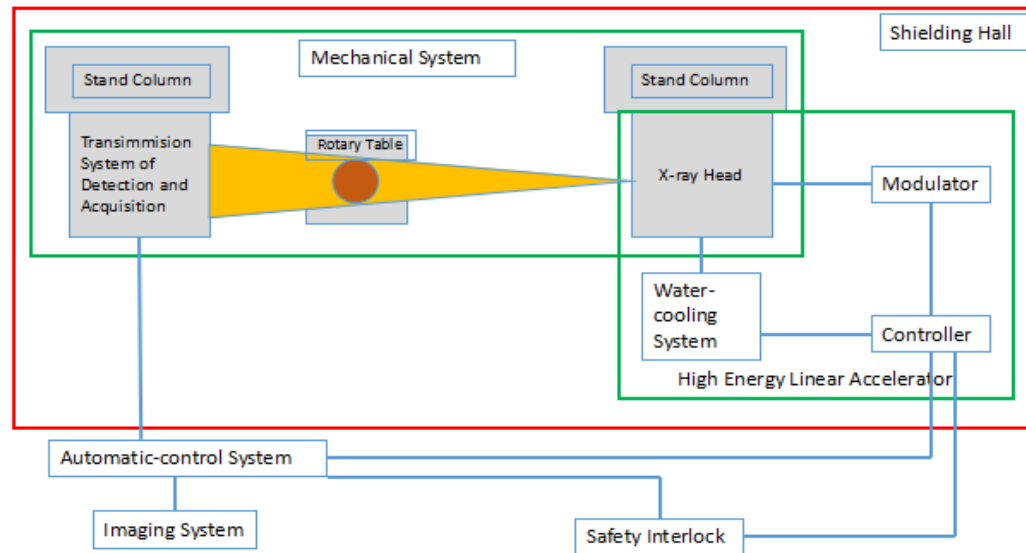
P10

ZHANG Xiang-yang

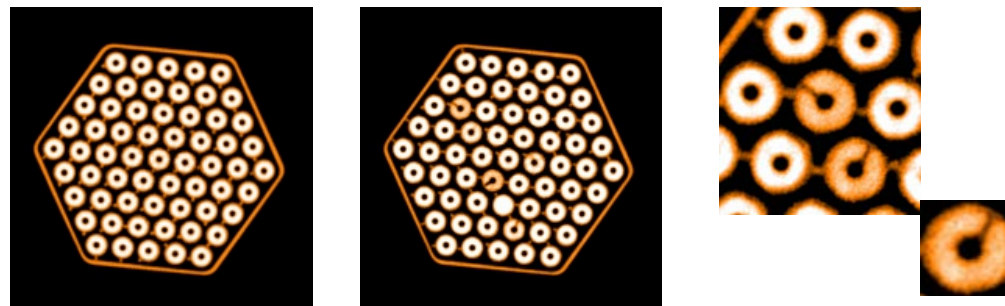
China Institute of Atomic Energy

High Energy X-ray Study on Nondestructive
Detection of Fuel Assemblies

Nuclear fuel assembly is the core of the reactor , in order to further study the performance of the fuel assemblies , we have conducted nondestructive detection research of assembly with high energy X-ray for several years, and experimental detection device was developed.



Schematic diagram of system structure



Computed tomography image of simulated nuclear fuel assembly



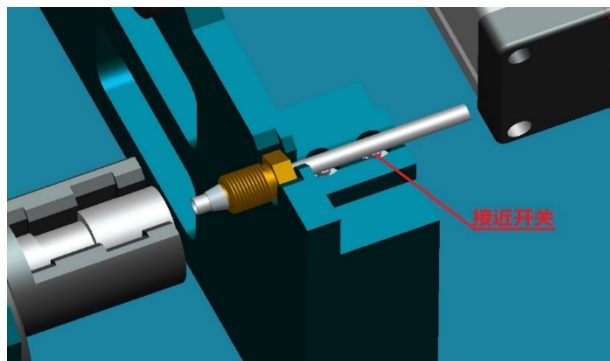
P11

Lei yang

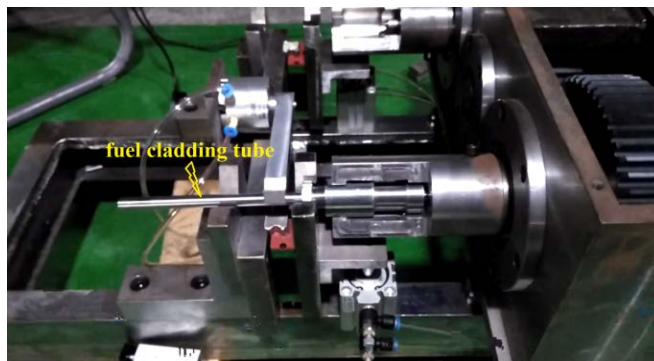
Nuclear Power Institute of China

Research on Closed-end Burst Testing of
Irradiated Fuel Cladding Tube

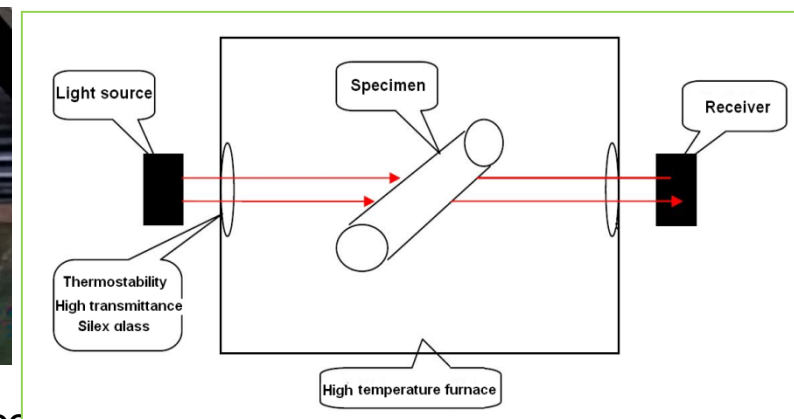
Research on Closed-end Burst Testing of Irradiated Fuel Cladding Tube



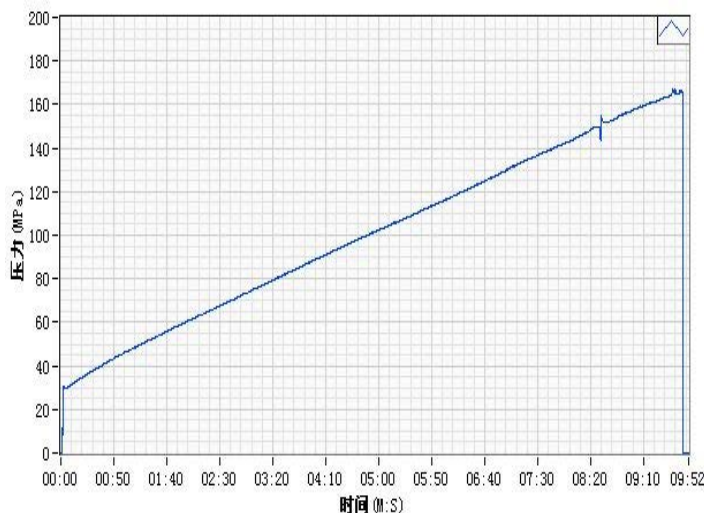
Designs for automatic loading



Automatic sealing and locking device

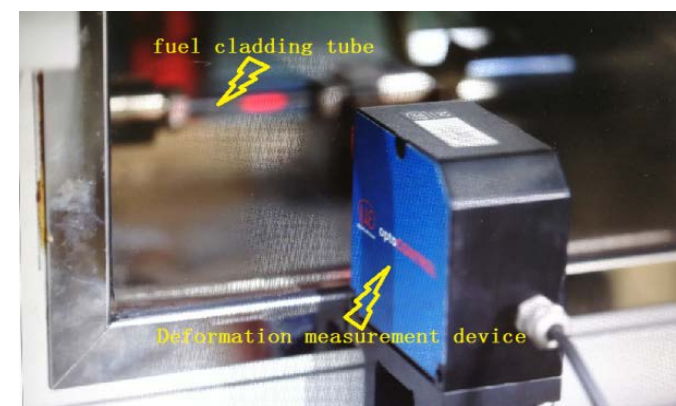


Measuring principle



The curve of pressure-time

Blasting pressure :
over 200MPa
Measurement accuracy :
5 μ m



Continuous deformation
measurement device

P12

Guo li-na

China Institute of Atomic Energy

The research on oxide dispersion strengthened (ODS) ferritic steel by chemical method



P13

Kiho Kim

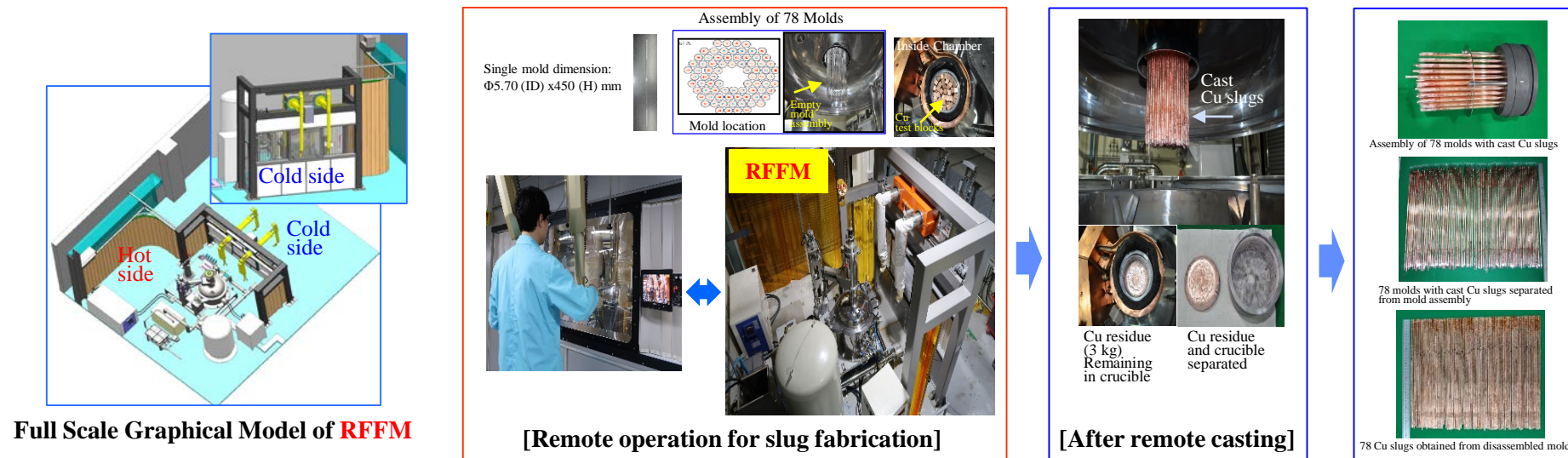
Korea Atomic Energy Research Institute

Remote Metal Fuel Slug Fabrication System Based
on Injection Casting

Remote Metal Fuel Slug Fabrication System Based on Injection Casting

Kiho KIM and Jeong-Yong PARK
Korea Atomic Energy Research Institute

- ◆ The Korea Atomic Energy Research Institute (KAERI) has been researching a technology for fabricating TRU (Transuranium) metal fuel using TRU ingot produced from Pyroprocessing.
- ◆ Such TRU metal fuel fabrication processes should be conducted in a fully remote manner at a hot-cell because of a nature of a radioactivity of TRU ingot. All the in-cell processes and equipment operations should be fully conducted using remote handling tools.
- ◆ To remote fuel fabrication technology development, we constructed the Remote Fuel Fabrication Mock-up (RFFM) located in the Fuel and Material Test Facility at KAERI this year.
- ◆ RFFM is an efficient means to test and verify an engineering-scale metal fuel fabrication using non-radioactive materials from the remote operation and maintenance viewpoint in advance before TRU ingots are used at hot-cell.
- ◆ RFFM consists of an engineering-scale metal fuel slug fabrication system and a remote handling system. The engineering-scale metal fuel slug fabrication system is one to fabricate metal slugs based on an injection casting. The remote handling system is a means to make the metal fuel slug fabrication possible in a remote manner.
- ◆ We demonstrate the constructed RFFM and its capability examined through the remote fabrication using copper conducted in RFFM. Remote copper slug fabrication results based on injection casting are also presented.





P14

Miho Suzuki

Japan Atomic Energy Agency

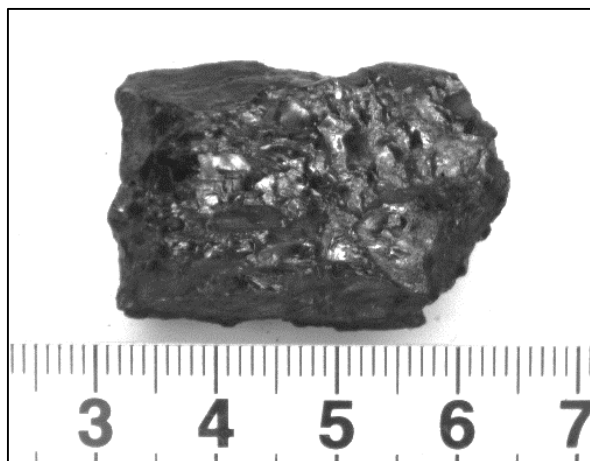
Sample preparation techniques for post
irradiation examinations in the Reactor Fuel
Examination Facility

M.Suzuki, Y.Kimura, M.Takano, N.Mita



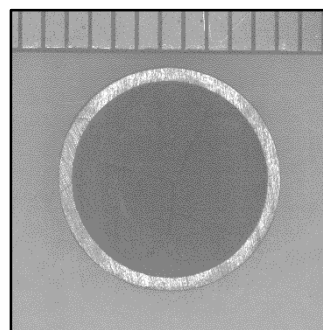
① Analysis for **TMI-2 debris**

How to hold the various shaped sample?

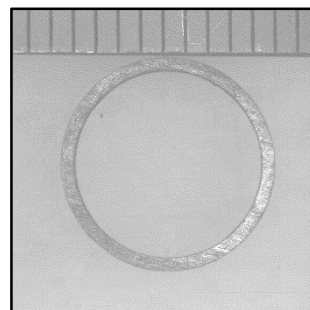


② **Precise hydrogen analysis** of cladding tube

How to remove the pellet without
damaging the cladding tube?

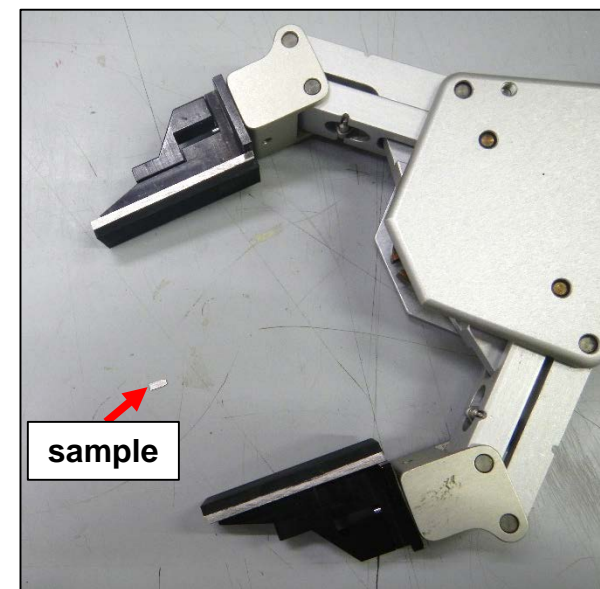


De-fueling



③ **SEM observation of small sample**

How to stand a
fragile sample?



**Please visit Poster
P14**



P15

Ryan Devlin

National Nuclear Laboratory

Development of Laser Ablation Inductively
Coupled Plasma – Mass and Optical Emission
Spectrometry Methodologies for Elemental
Analysis in a Medium Active Cell Environment

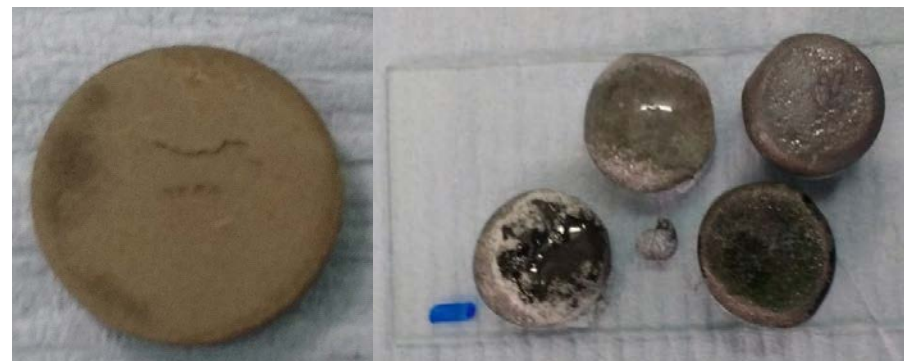
Development of Laser Ablation Inductively Coupled Plasma – Mass and Optical Emission Spectrometry Methodologies for Elemental Analysis in a Medium Active Cell Environment

Ryan Devlin¹, Clive Lythgoe² & Simon Chenery³

¹National Nuclear Laboratory, Central Laboratory, Sellafield, Seascale, CA20 1PG, UK

²Sellafield Ltd, Sellafield, Seascale, CA20 1PG, UK

³British Geological Survey, Environmental Science Centre, Keyworth, Nottingham, NG12 5GG, UK



**British
Geological Survey**
Expert | Impartial | Innovative

**NATIONAL NUCLEAR
LABORATORY**

P16

Anabelle Lopez

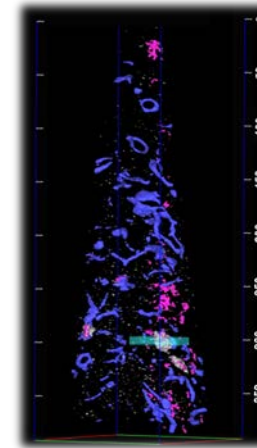
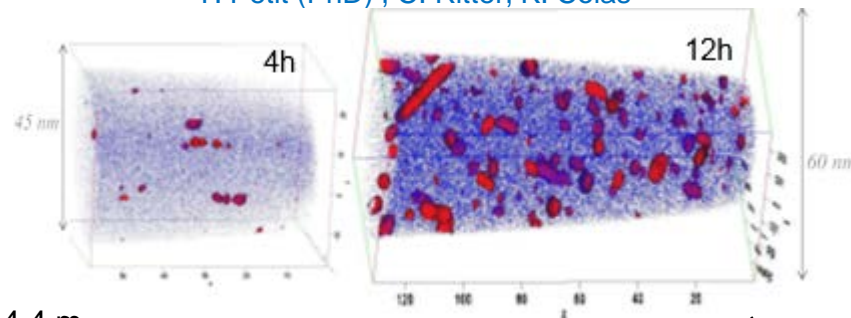
*CEA-DEN, Service d'Études des Matériaux Irradiés,
CEA, Université Paris-Saclay*

Nuclearization projects of a tomographic atom probe
and of an electropolishing machine for researches on
neutron irradiated materials at the atomic scale

NUCLEARIZATION OF A TOMOGRAPHIC ATOM PROBE AND AN ELECTROPOLISHING MACHINE FOR NEUTRON IRRADIATED MATERIAL STUDIES AT THE NANOMETRIC SCALE IN CEA/PARIS-SACLAY LECI HOTLAB FACILITY

Aluminum alloy (structural hardening) : MgSi clusters

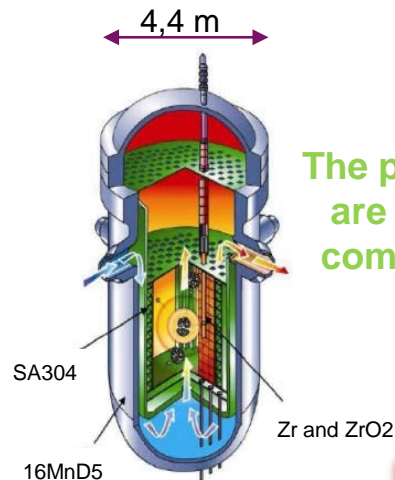
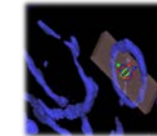
T. Petit (PhD) , C. Ritter, K. Colas



304L Stainless steel
irradiated with Fe ions :

Segregation around
dislocation loops

E. Paccou (PhD), B. Tanguy



The phenomena controlling materials behavior under irradiation are on the atomic scale → Impact mechanical behavior at the components scale → Essential to understand ageing of these materials and to improve the nuclear safety. So how?



With a **nuclearized** Tomographic Atom Probe and **nuclearized** sample preparation methods



anabelle.lopez@cea.fr
kimberly.colas@cea.fr

Nota : This work profited from a French government grant managed by the National Agency of Research under the program "Investments for the future" (ref. ANR-11-EQPX-0020)

P17

Olivier Dugne

*CEA, Nuclear Energy Division, Research Department
on Mining and Fuel Recycling Processes*

Nuclearized Raman microscope coupled with a hot-stage:
new tool to study (U,Pu)O_{2-x} fuel microstructure

New Raman microscope setup: L26, ATALANTE facility, France

Horiba© IHR-320 Raman spectrometer with 2 lasers: 532 nm & 660 nm

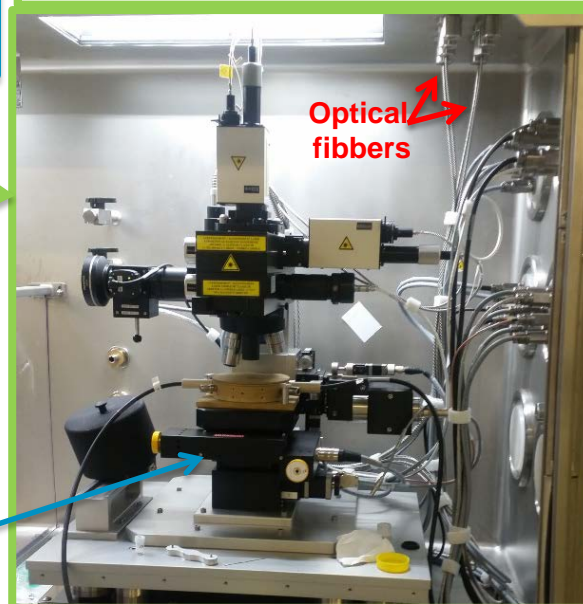
Confocal microscope equipped with 2 exchangeable turrets dedicated to:

- **Ex situ** microstructural observations (optical & Raman): equipped with 4 Olympus© objectives (x5 to x100), the micro-indenter device and a Raman calibration objective.
- **In situ** high temperature Raman experiment: equipped with 3 Mitutoyo© long working distance objectives (x5 to x50) and a Raman calibration objective.

Glovebox dedicated to μ -Raman

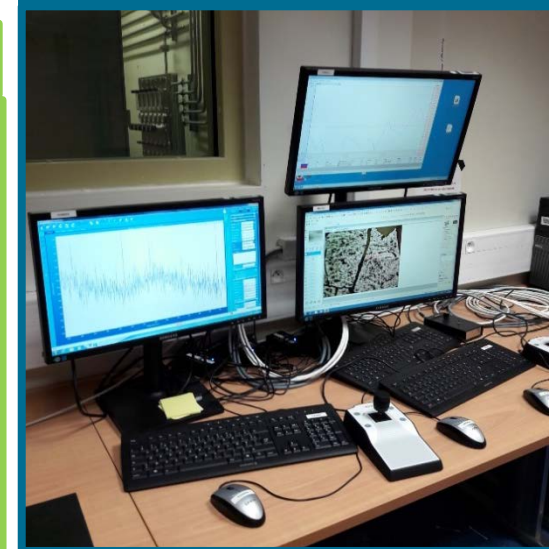


Confocal microscope



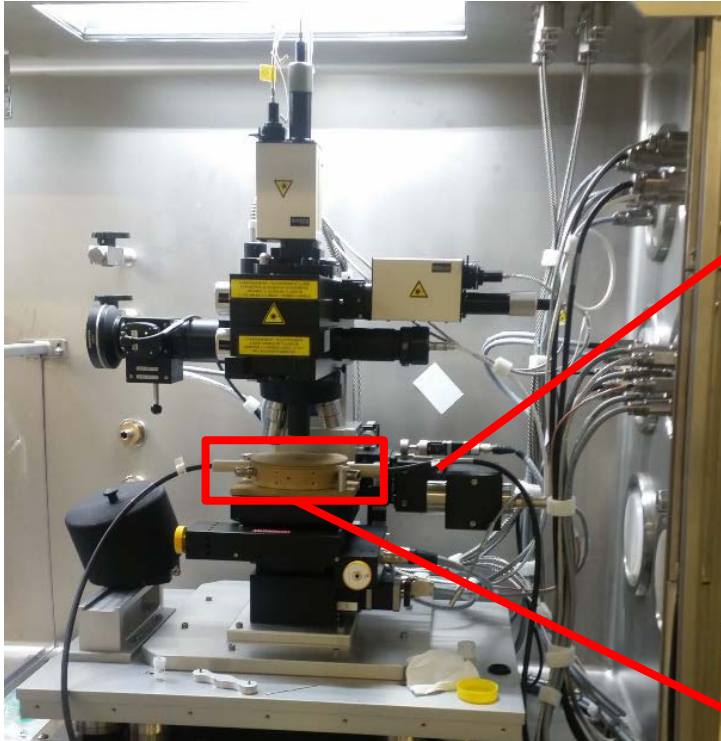
Motorized stages : x, y, z and sample rotation

Remote control system



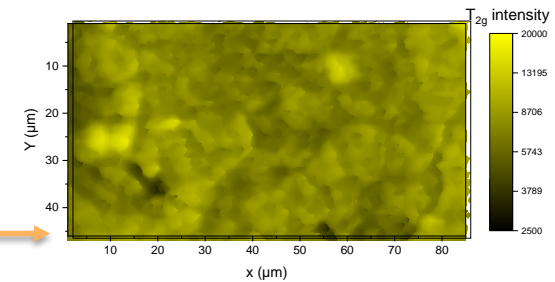
New Raman microscope setup: L26, ATALANTE facility, France

Ex situ experiments

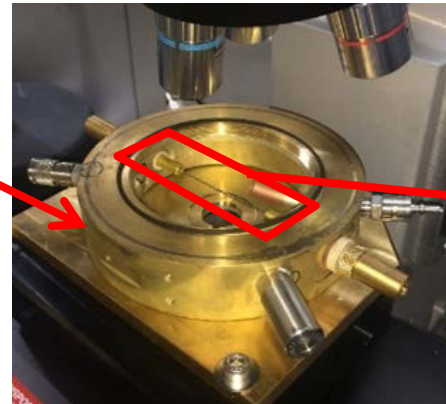


- Imaging (optical & Raman)
- μ -Hardness (Vickers)

$(\text{Ce}_{0.85}\text{Y}_{0.15})\text{O}_{2-x}$ Raman map



In situ experiments



Heating wire

- Up to 2000°C
- Controlled pO_2
- Powders
- Sintered samples (1.5x2 mm²)

See dedicated poster for details and illustrations

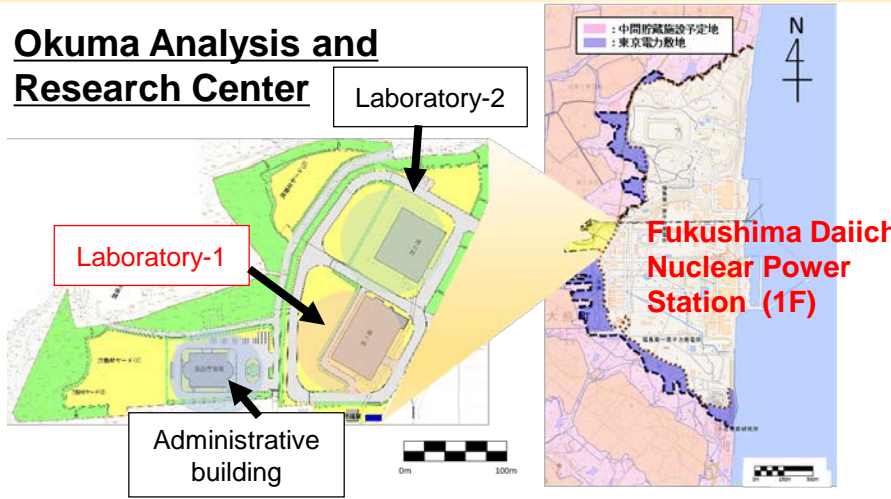
P18

Maho Iwasaki

Japan Atomic Energy Agency

Application of ICP-MS to analysis of nuclear fuel
debris and radioactive wastes

Okuma Analysis and Research Center



Our missions

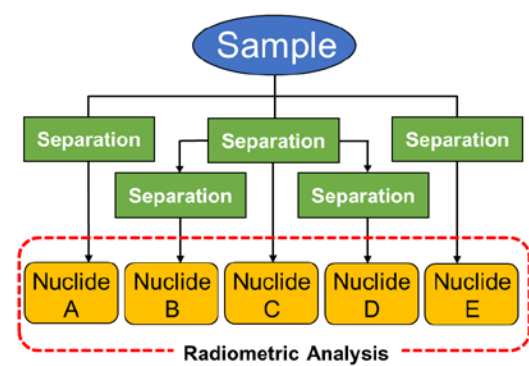
Set up facilities supporting the decommissioning of 1F.

Perform analysis of radioactive waste samples and fuel debris from 1F.

Samples are measured at laboratory-1 and laboratory-2.

At the beginning: 200 samples/year, 38 nuclides.

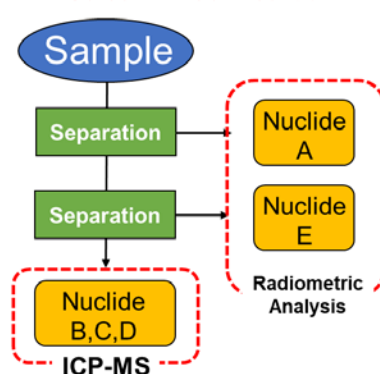
Conventional methods



- ☑ Complicated handling
- ☑ Time-consuming

Low analysis capability

Streamlined method

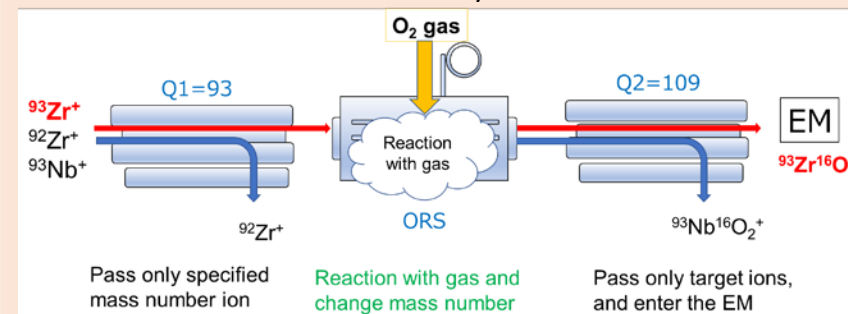


- ☑ Easier handling
- ☑ Shorter analysis time

High analysis capability

ICP-QQQ-MS(Agilent 8900)

Demonstration of Zr selective analysis



Q1: Quadrupole 1 (The first mass filter)
Q2: Quadrupole 2 (The second mass filter)
ORS: Octapole Reaction Cell System, collision/reaction cell
EM: Electron multiplier detector

Regarding Zr measurement, it is possible to eliminate the isobaric interferences using ICP-QQQ-MS with suitable "reaction gasses, NH₃ in He".

P19

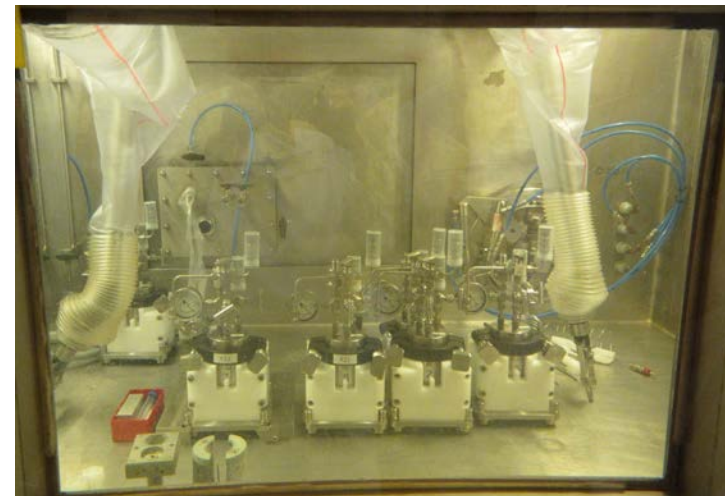
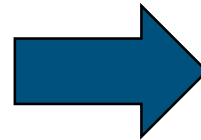
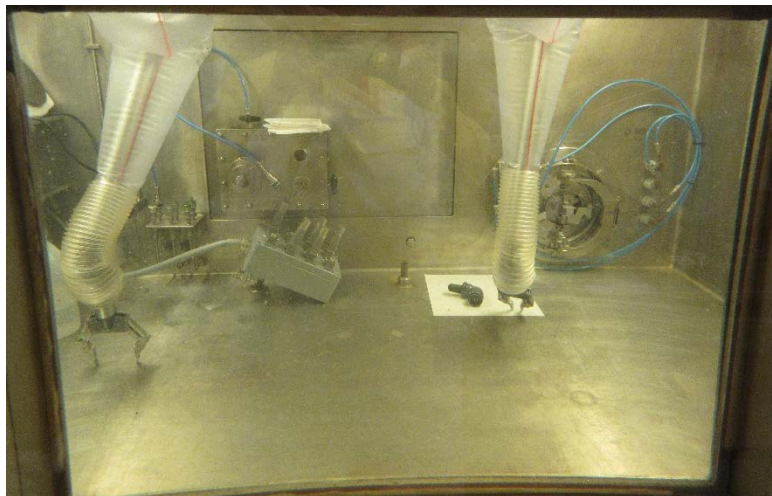
Guy Cornelis

Nuclear Materials Science Institute, SCK•CEN

Design, development, and installation of hot cell
instrumentation for Spent Fuel Autoclave
Leaching Experiments (SF-ALE)

Radionuclide release from spent nuclear fuel under geological repository conditions is required for realistic safety assessment

- The FIRST-nuclides project with a primitive set-up, 2012 - 2015
- Fuel dissolution under fully controlled conditions, 2017 →



- How, when, why? See the poster for details!

P20

Stéphane Brémier

*European Commission, Joint Research Centre, JRC-
Directorate G – Nuclear Safety and Security*

The Decommissioning and Waste Management
programme of the European Commission Joint
Research Centre

The Decommissioning and Waste Management programme of the European Commission Joint Research Centre

The programme aims to eliminate both historical and future liabilities at all nuclear sites of the JRC

JRC-Petten



High Flux Reactor
Auxiliary buildings &
infrastructure

Since 1961

JRC-Geel



Accelerators Since 1965
Glove box laboratories,
effluent water systems

JRC-Ispra

Since early 60s

2 Reactors &
ancillary labs



JRC-Karlsruhe



Hot Cells,
commercial and
'exotic' spent
fuels

Glove box
laboratories

Since 1965



Hot cells
facility



Waste
management



Liquid effluents, Cyclotron, legacy
wastes...



The JRC Nuclear Decommissioning and Waste Management Programme was formally launched in 1999 (COM(1999)114, Council and European Parliament).

P21

Petr Švrčula

Research Centre Rez Ltd.

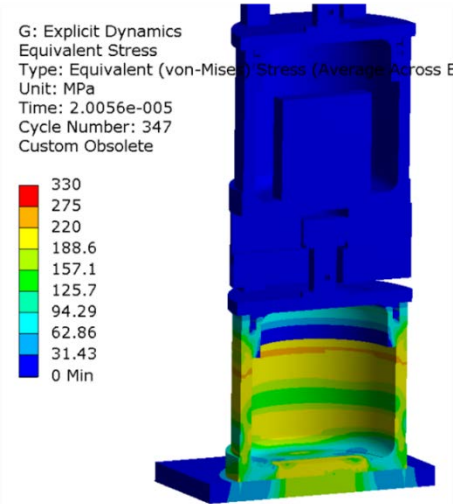
Transportation capabilities of hot cell facility

Transportations capabilities of the hot cell facility

Petr Švrčula, Ondřej Srba, Maria Zimina



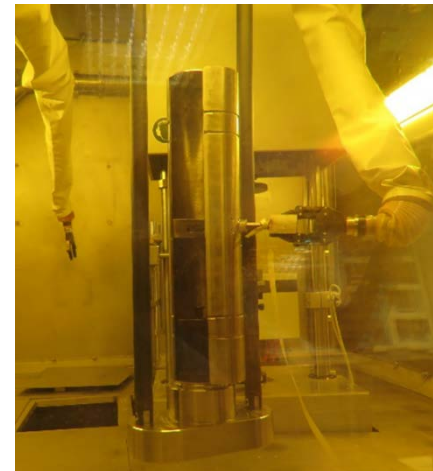
- Transport cask TERA 300 type B(U)
- Domestic and international transportation
- Shielding abilities 300 TBq of ^{60}Co
- Horizontal and vertical loading



Impact deformation calculation



Capsule model



Inner canal operated in cell



TERA 300



P22

Amir Hushyar

Transnubel, Dessel, Belgium

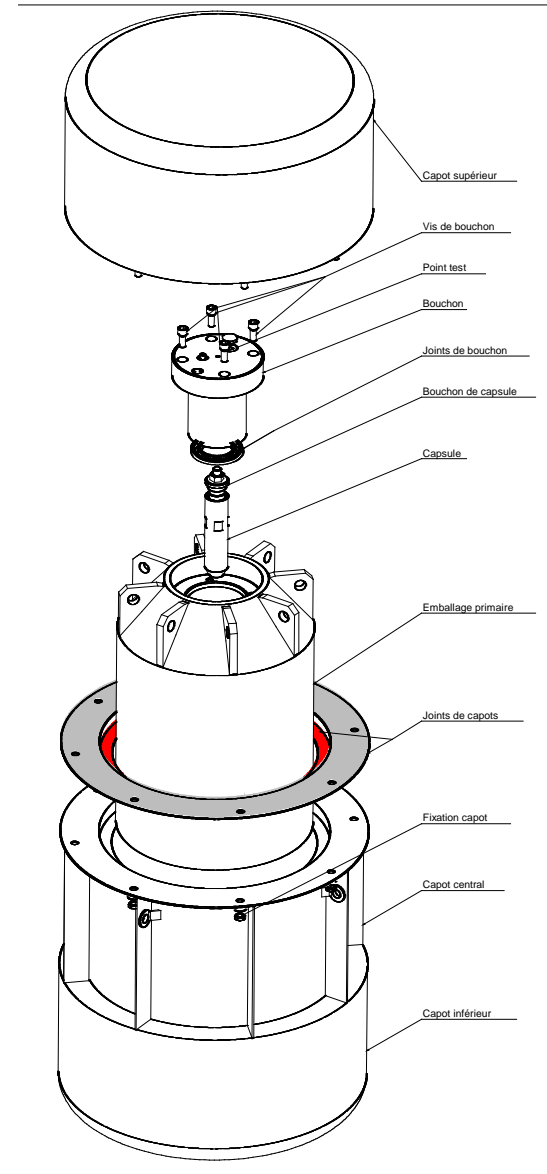
First transport campaign of new type B(U)
packaging for hotlabs

TNB 170

- Allowed contents
 - fresh or irradiated UOX/MOX fuel
 - sealed or unsealed radioactive sources
 - neutron sources of type Xx-Be
- FANC license B(U)
- H/V loading and unloading
- Tilting, docking et extension tools

Dimensions	Outer Dimensions	Inner Dimensions
Diameter	600 mm	48 mm
Length	921 mm	201 mm

Empty Weight	592 kg
Maximum Load	2,5 kg



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Sunggeun Kim

Korea Atomic Energy Research Institute

Mechanical test of spent fuel at KAERI-PIEF