

Post-Irradiation Annealing Studies of Small-Scale U-Mo Monolithic Fuel Samples

HOTLAB-2014

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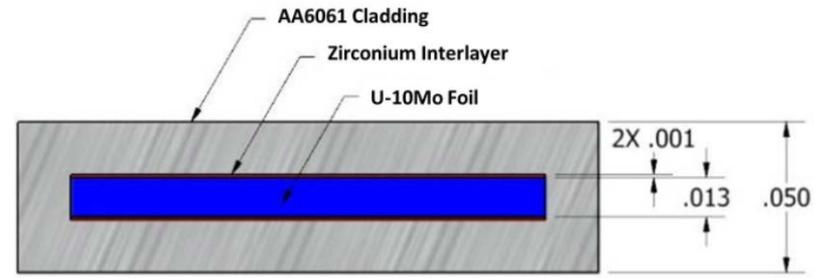
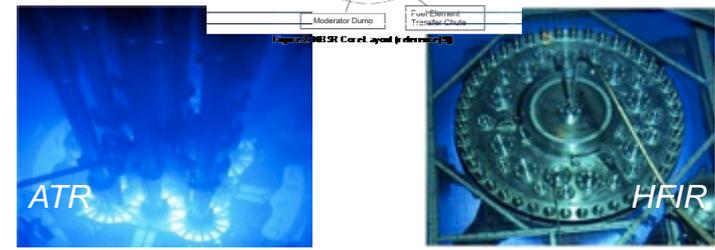
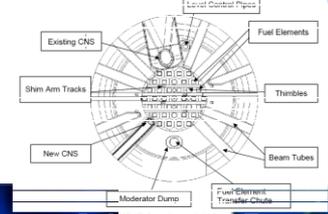
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Development of high density LEU fuel

- ▶ The Global Threat Reduction Initiative (GTRI) is supporting development of a high density LEU fuel
- ▶ The new fuel would support the conversion of as many as five US High Performance Research Reactors
- ▶ The fuel is based on a U-Mo alloy and consists of a monolithic foil design
- ▶ PNNL leads the fuel fabrication efforts for the GTRI and supports other aspects of the program, including physical property characterization of unirradiated and irradiated fuel samples



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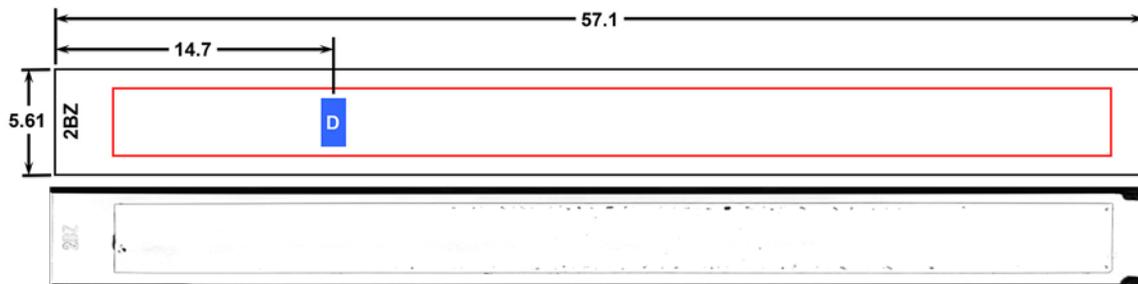
RPL Hotcell Capabilities

- ▶ Radiochemical Process Laboratory (RPL) is a Cat 2 facility with capabilities for:
 - Handling, characterizing, and cutting of irradiated fuel and structural materials
- ▶ A total of eight hotcells currently dedicated to thermal-physical property measurements of irradiated fuel
 - PDC1 – TG/DTA-MS utilized for small scale testing of fission product release as a function of time and temperature
 - PDC2 – LFA utilized to obtain thermal diffusivity as a function of temperature
 - ISC1 – Optical microscope utilized to obtain layer thicknesses and microstructural information
 - SAL2 – Pycnometer utilized to obtain density
 - SAL2,3,4 – Analytical chemistry digestion
 - SAL5 – Metallography preparation equipment (pot, grind, polish)
 - SAL6 – Sample sectioning equipment and DSC utilized to obtain heat capacity as a function of temperature



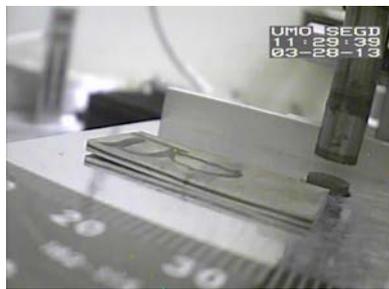
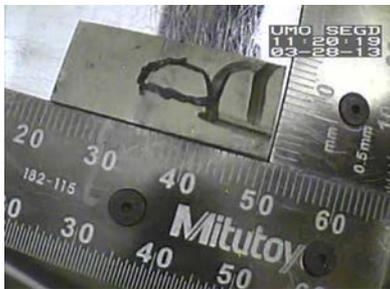
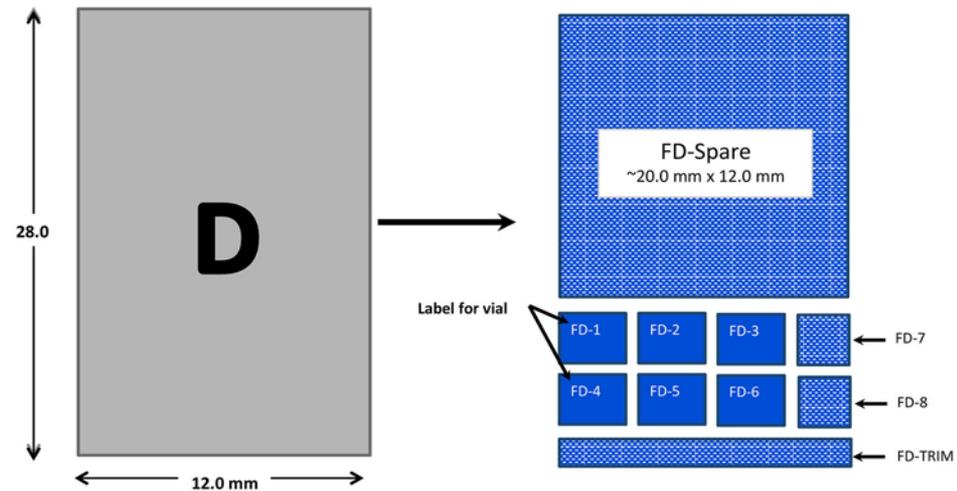
Experimental Materials

- ▶ Segment D was from AFIP-2BZ fuel plate
 - 132.4 EFPD in ATR CFT
 - Surface heat flux of $\sim 237 \text{ W}\cdot\text{cm}^{-2}$
- ▶ PNNL received a fuel segment $\sim 25 \text{ mm}$ long \times $\sim 13 \text{ mm}$ wide from INL
- ▶ Segment had an average burnup of $\sim 49\%$
- ▶ Segment was sectioned into smaller samples at PNNL for fission product release measurements

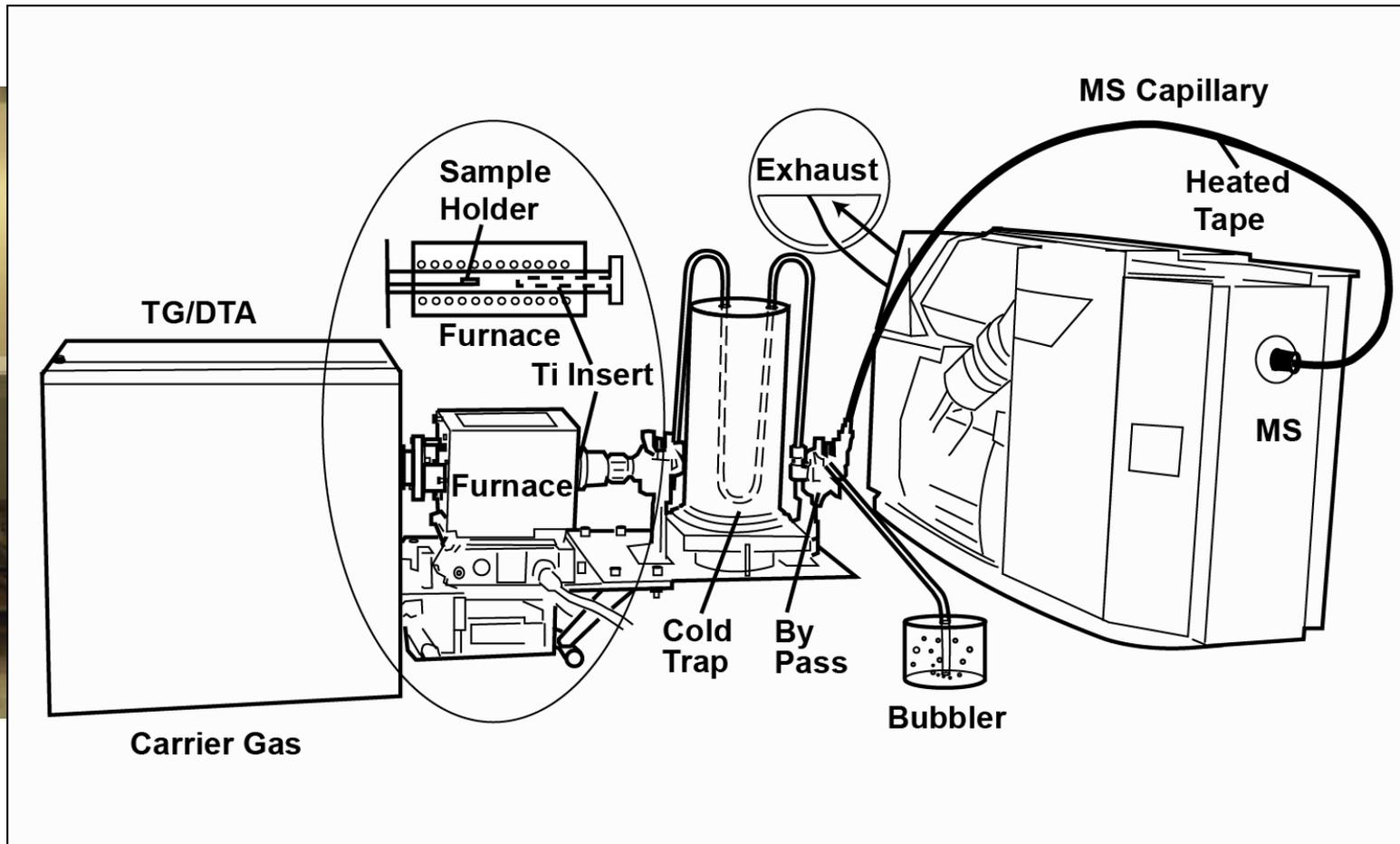


Segment D Samples for Fission Gas Analyses

■ = Samples
■ = Spare Samples

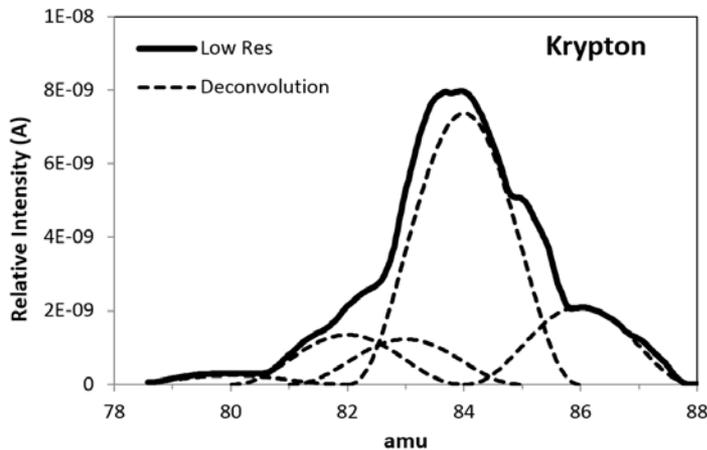


Equipment Set-up

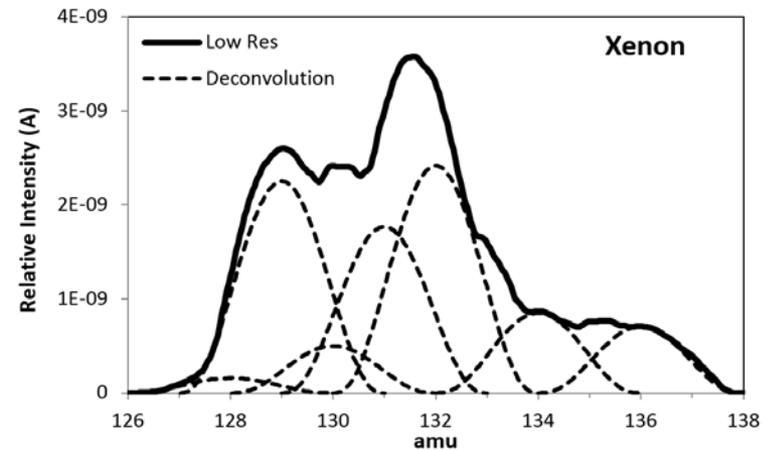


Peak Analysis

- ▶ Calibration gases analyzed for isotopic distribution to correlate concentration with a specific amu being measured
- ▶ Low resolution settings used due to small sample size and low heating rate
 - Increased the amount of data acquired per unit of elapsed time
 - Required deconvolution as a result of peak broadening and overlap
- ▶ Measured peaks within 3% of natural abundance isotopic values



Stable Kr Isotopes amu	Natural Abundance %	Measured %
80	2.3	2.0
82	11.6	11.0
83	11.5	10.0
84	57.0	59.8
86	17.3	17.2

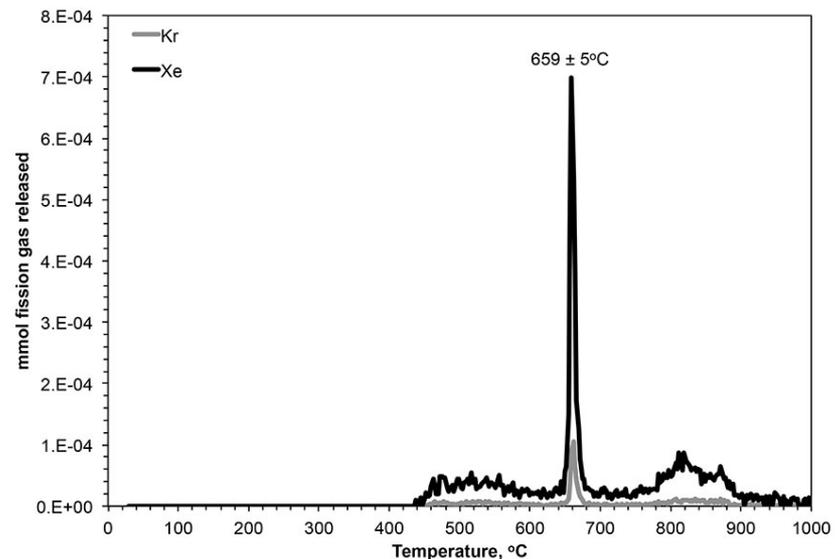
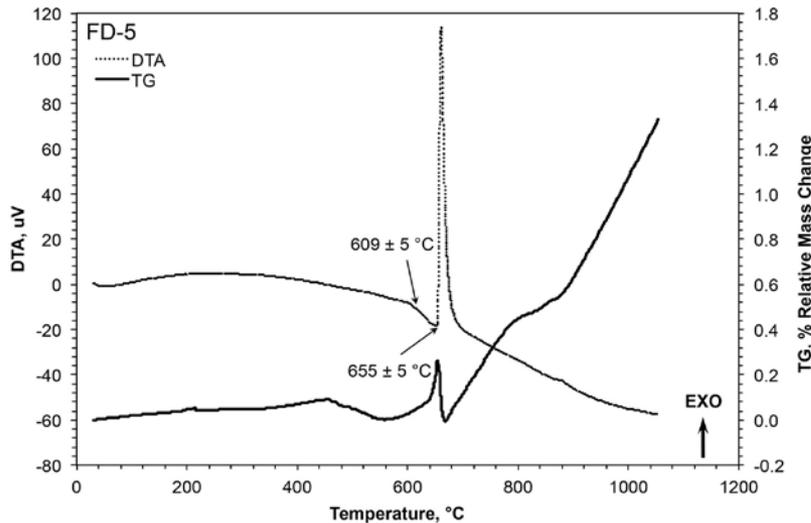
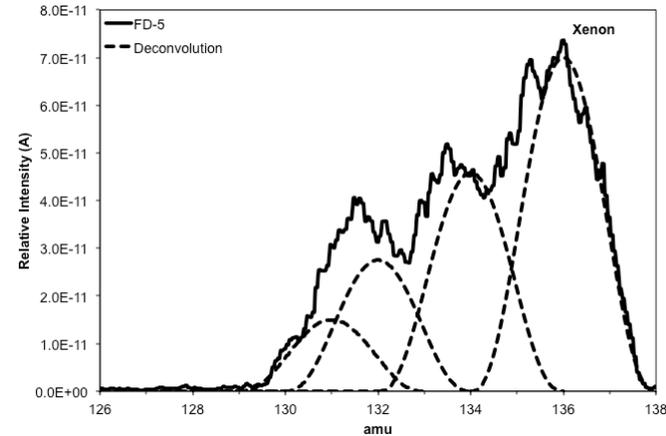
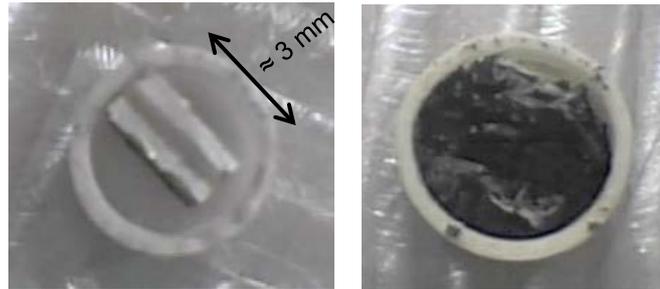


Stable Xe Isotopes amu	Natural Abundance %	Measured %
128	1.9	1.8
129	26.4	26.0
130	4.1	5.8
131	21.2	20.4
132	26.9	27.9
134	10.4	9.8
136	8.9	8.2

Irradiated Sample FD-5

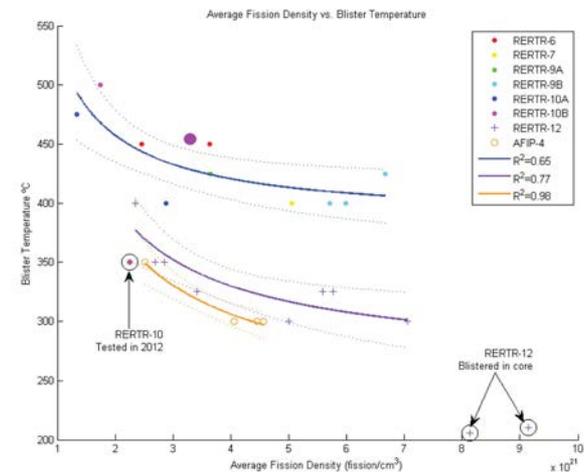
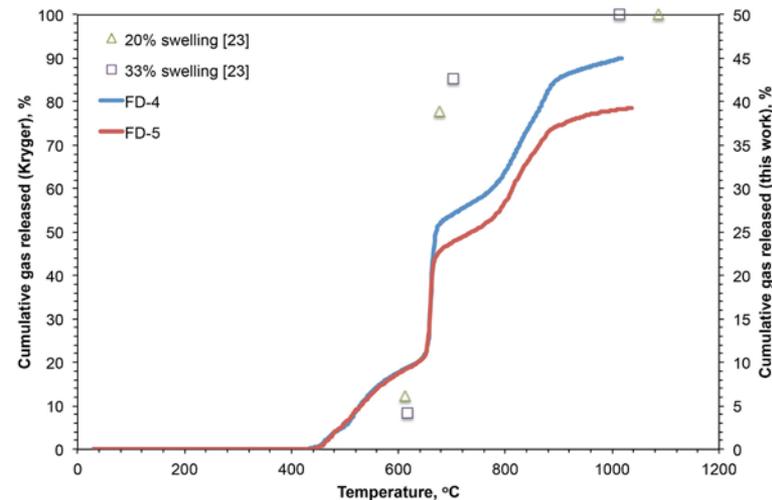
Due to small sample size the system was optimized by decreasing the resolution and scan rate

- More data points over the thermal cycle (lower °C/scan)
- Enhanced sensitivity through increase in ion current signal by a factor of:
 - ≈ 20 for Kr
 - ≈ 30 for Xe



Fission Gas Release Mechanisms

- ▶ First release at 455 °C
 - Precipitation of gaseous fission products from solid solution
 - Coalescence into large pores that escape through exposed edges of sample
- ▶ Second release at ~640 °C
 - Onset of cladding melt at 654 °C
 - Exothermic reaction between liquid Al and solid Zr at 654 °C
 - Massive swelling of fuel and loss of cladding constraint
- ▶ Third release between 770 and 885 °C
 - Melt temperature of U-10Mo is ~1140 °C
 - Homologous temperature is typically 2/3 of melt temperature
 - Lattice diffusion makes significant contribution to solid-state diffusion at this temperature
- ▶ Additional studies to deduce whether all of FG was released, or whether residual remains in sample is underway



Meyer et al., INL/EXT-12-26500, June 2012, p. 3

Summary

- ▶ A variety of thermal property measurement equipment has been installed into hot cells at the Pacific Northwest National Laboratory
- ▶ Small scale tests to evaluate fission product release as a function of temperature have been performed and optimized
- ▶ The equipment is very capable of performing the desired measurements and the processes / procedures that have been put in place are adequate for obtaining FG release data on small samples
- ▶ Initial measurements reveal three distinct FG releases for clad monolithic U-Mo samples
- ▶ Data correlates well with historical measurement, but full release was not observed
- ▶ Additional measurements are currently underway to evaluate correlations used to calculate FG release and residual FG in the sample